

Dated at Rockville, Maryland this 23rd day of January 2001.

For the Nuclear Regulatory Commission.

Patrick D. Milano,

Senior Project Manager, Section 1, Project Directorate 1, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 01-2480 Filed 1-26-01; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket 72-10]

Nuclear Management Company, LLC; Prairie Island Independent Spent Fuel Storage Installation; Issuance of Environmental Assessment and Finding of No Significant Impact Regarding the Proposed Amendment To Revise the License and Technical Specifications of License No. SNM-2506

The U.S. Nuclear Regulatory Commission (NRC or Commission) is considering issuance of an amendment, pursuant to 10 CFR 72.56, to the Special Nuclear Materials License No. 2506 (SNM-2506) held by the Nuclear Management Company, LLC (licensee) for the Prairie Island independent spent fuel storage installation (ISFSI). The requested amendment would revise the license and Technical Specifications of SNM-2506 to specifically permit the storage of burnable poison rod assemblies (BPRAs) and thimble plug devices (TBDs) within the TN-40 casks used at the Prairie Island ISFSI.

Environmental Assessment (EA)

Identification of Proposed Action

The staff is considering issuance of an amendment to revise the license and Technical Specifications of SNM-2506 for the Prairie Island ISFSI. The changes to the license and Technical Specifications would specifically permit the storage of BPRAs and/or TPDs within the TN-40 dry storage casks used at the Prairie Island ISFSI.

Need for the Proposed Action

The proposed action is needed to eliminate the need to physically remove BPRAs and TPDs from irradiated fuel assemblies that have already been loaded into the TN-40 dry storage casks and irradiated fuel assemblies that will be loaded into TN-40 dry storage casks in the future. Permitting the proposed action would result in the reduction of exposure time to plant workers handling the BPRAs and TPDs and a more effective ALARA program pursuant to 10 CFR 20.1101(b).

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that granting the amendment to specifically allow the storage of BPRAs and TPDs within the TN-40 casks used at the Prairie Island ISFSI will not increase the probability or consequences of accidents. No changes are being made in the types of any effluents that may be released offsite. With regard to radiological impacts, the addition of irradiated BPRAs and TPDs only affects the gamma source term of the cask. The offsite dose rates were calculated to increase an insignificant amount. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

The amendment only affects the requirements associated with the content of the casks and does not affect non-radiological plant effluents or any other aspects of the environment. Therefore, there are no significant non-radiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Alternative to the Proposed Action

Since there is no significant environmental impact associated with the proposed action, alternatives are not evaluated other than the no action alternative. The alternative to the proposed action would be to deny the request for amendment (i.e., the "no-action" alternative). Denial of the proposed action would result in greater exposures to plant workers due to the fact that the BPRAs and TPDs would have to be physically removed from each fuel assembly possessing them. Physical removal of irradiated BPRAs and TPDs would increase exposure time and dose to the plant workers and would require disposal or storage of additional radioactive material (i.e., BPRAs and TPDs) that would otherwise be safely stored if the BPRAs and TPDs are left intact with its irradiated fuel assembly and loaded into dry cask storage. The environmental impacts of the alternative action may be greater than the proposed action.

Given that there may be greater environmental impacts associated with the alternative action of denying the amendment, the Commission concludes that the preferred alternative is to grant this amendment request.

Agencies and Persons Consulted

On December 27, 2000, Mr. Rakow of the Minnesota Department of

Commerce, Electric Unit, was consulted about the EA for the proposed action and had no concerns.

Finding of No Significant Impact

The environmental impacts of the proposed action have been reviewed in accordance with the requirements set forth in 10 CFR part 51. Based upon the foregoing EA, the Commission finds that the proposed action of granting an amendment to permit the storage of BPRAs and TPDs within the TN-40 casks used at the Prairie Island ISFSI will not significantly impact the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to this action, see the amendment application dated August 31, 1999, as supplemented. These documents are available for public inspection at the Commission's Public Document Room, 11155 Rockville Pike, Rockville, MD.

Dated at Rockville, Maryland, this 21st day of January 2001.

For the Nuclear Regulatory Commission.

E. William Brach,

Director, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 01-2481 Filed 1-26-01; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Nuclear Waste Working Group Meeting; Notice of Meeting

The ACNW Working Group meeting will hold a meeting on February 21-22, 2001, Room T-2B1, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, February 21, 2001—8:30

a.m. until the conclusion of business

Thursday, February 22, 2001—8:30 a.m. until 12:00 Noon

The Working Group will review the chemistry aspects of documents intended to support the Yucca Mountain Site Recommendation Considerations Report (SRCR) and the NRC/DOE issue resolution process. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the

concurrence of the Working Group Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Working Group, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACNW staff scientist named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Working Group, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Working Group may then hear presentations by and hold discussions with representatives of the NRC staff, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor, can be obtained by contacting the cognizant ACNW staff scientist, Dr. Andrew C. Campbell (telephone 301/415-6897) between 8:30 a.m. and 4:30 p.m. (EST). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: January 22, 2001.

James E. Lyons,

Associate Director for Technical Support, ACRS/ACNW.

[FR Doc. 01-2477 Filed 1-26-01; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Subcommittee on Plant License Renewal; Notice of Meeting

The ACRS Subcommittee on Plant License Renewal will hold a meeting on February 22, 2001, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Thursday, February 22, 2001—8:30 a.m. until the conclusion of business

The Subcommittee will discuss the NRC staff's draft Safety Evaluation

Report for the Entergy Operations, Inc., license renewal application for Arkansas Nuclear One, Unit 1 and related matters. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of the NRC staff, Entergy Operations, Inc., and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor, can be obtained by contacting the cognizant ACRS staff engineer, Mr. Sam Duraiswamy (telephone 301/415-7364) between 7:30 a.m. and 4:15 p.m. (EST). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: January 22, 2001.

James E. Lyons,

Associate Director for Technical Support, ACRS/ACNW.

[FR Doc. 01-2478 Filed 1-26-01; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Joint Meeting of the ACRS Subcommittees on Plant Operations and on Reliability and Probabilistic Risk Assessment; Notice of Meeting

The ACRS Subcommittees on Plant Operations and on Reliability and Probabilistic Risk Assessment will hold a joint meeting on February 21, 2001, in Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, February 21, 2001—8:30 a.m. until the conclusion of business

The Subcommittees will discuss the South Texas Project Nuclear Operating Company's exemption request to exclude certain components from the scope of special treatment requirements in 10 CFR parts 21, 50, and 100. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairmen and written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittees, their consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittees, along with any of their consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittees will then hear presentations by and hold discussions with representatives of the NRC staff, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefore, can be obtained by contacting the cognizant ACRS staff engineer, Ms. Maggalean W. Weston (telephone 301/