OMB Reviewer: Alexander T. Hunt, (202) 395–7860, Office of Management and Budget, Room 10226, New Executive Office Building, Washington, DC 20503.

FOR FURTHER INFORMATION CONTACT: Copies of the information collection requests, with applicable supporting documentation, may be obtained by calling the NCUA Clearance Officer, C. Keith Morton, (703) 518–6411.

SUPPLEMENTARY INFORMATION: Proposals for the following collection of information:

OMB Number: 3133–0114.

Form Number: N/A.

Type of Review: Extension of a currently approved collection.

Title: Payment on Shares by Public Units and Nonmembers

Description: 5 CFR 701.32 limits nonmember and public unit deposits in federally insured credit unions to 20 percent of their shares or $1.5 million, whichever is greater. The collection of information requirement is for those credit unions seeking an exemption from the above limit.

Respondents: Credit Unions seeking an exemption from the limits on share deposits by public unit and nonmember accounts set by 5 CFR 701.32.

Estimated No. of Respondents/ Recordkeepers: 20.

Estimated Burden Hours Per Response: 2 hours.

Frequency of Response: Other. As exemption is requested.

Estimated Total Annual Burden Hours: 40.

Estimated Total Annual Cost: N/A.

By the National Credit Union Administration Board on August 30, 2001.

Hattie Ulan,

Acting Secretary of the Board.

[FR Doc. 01–22298 Filed 9–5–01; 8:45 am]

BILLING CODE 7535–01–P

NATIONAL FOUNDATION ON THE ARTS AND THE HUMANITIES

National Endowment for the Arts; Leadership Initiatives Advisory Panel

Pursuant to Section 10(a)(2) of the Federal Advisory Committee Act (Public Law 92–463), as amended, notice is hereby given that a meeting of the Leadership Initiatives Advisory Panel (AccessAbility section) will be held by teleconference from 2 p.m. to 4 p.m. on Wednesday, September 26, 2001 in Room 528 at the Nancy Hanks Center, 1100 Pennsylvania Avenue, NW., Washington, DC 20506.

This meeting is for the purpose of review, discussion, evaluation, and recommendations on financial assistance under the National Foundation on the Arts and the Humanities Act of 1965, as amended, including information given in confidence to the agency. In accordance with the determination of the Chairman of May 22, 2001, these sessions will be closed to the public pursuant to subsection (c)(4), (6) and (9)(B) of section 552b of Title 5, United States Code.

Further information with reference to this meeting can be obtained from Ms. Kathy Plowitz-Worden, Panel Coordinator, National Endowment for the Arts, Washington, DC 20506, or call 202/682–5691.

Dated: August 31, 2001.

Kathy Plowitz-Worden,

Panel Coordinator, Panel Operations, National Endowment for the Arts.

[FR Doc. 01–22388 Filed 9–5–01; 8:45 am]

BILLING CODE 7537–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–346]

Davis-Besse Nuclear Power Station; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF–3 issued to FirstEnergy Nuclear Operating Company (FENOC) for operation of the Davis-Besse Nuclear Power Station (DBNPS), Unit 1, located in Ottawa County, Ohio.

The proposed amendment would change Technical Specification (TS) Sections 3/4.9.7, Refueling Operations—Crane Travel—Fuel Handling Building, and associated Bases; TS 3/4.9.11, Refueling Operations—Storage Pool Water Level, and associated bases; TS 3/4.9.12, Refueling Operations—Storage Pool Ventilation; TS 3/4.9.13, Refueling Operations—Spent Fuel Assembly Storage, and associated Bases; and TS 5.6 Design Features—Fuel Storage. The purpose of this license amendment application is to propose the necessary revisions to the DBNPS TS to reflect an increase in Spent Fuel Pool (SFP) storage capability, as a result of the SFP re-racking project, from the current capacity of 735 fuel assemblies to a new capacity of 1624 fuel assemblies. To provide additional temporary storage of fuel assemblies to support a complete re-racking of the SFP, this license amendment application also requests approval for up to 90 transfer pit storage locations. The transfer pit storage rack will be relocated into the SFP as part of the completion of this re-racking project. The resulting SFP fuel storage capacity will be sufficient to meet storage needs through the current expiration date of the DBNPS operating license, April 22, 2017.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below. The Davis-Besse Nuclear Power Station (DBNPS) has reviewed the proposed changes and determined that a significant hazards consideration does not exist because operation of the DBNPS, Unit No. 1, in accordance with these changes would:

1a. Not involve a significant increase in the probability of an accident previously evaluated because the methods and procedures for handling fuel assemblies will remain unchanged, fuel handling equipment reliability will be unaffected, and provisions will remain in place to ensure that the likelihood of a heavy load drop will remain extremely small. The proposed changes involve an expanded SFP storage capacity resulting from the planned re-racking of the SFP, and the inclusion of provisions allowing for temporary storage of fuel assemblies in the transfer pit.

For the installation activities involving the proposed expanded spent fuel storage capacity, heavy load lifts have been given careful consideration. In accordance with the proposed changes to Technical Specifications (TS) 3/4.9.7, ''Crane Travel—Fuel Handling Building,'' except when a specially designed impact cover is placed over fuel assemblies located in the cask pit, heavy loads are prohibited from travel

over stored fuel assemblies. The physical design of the impact cover, together with administrative controls established while the impact cover is being installed or removed, ensure that it can not fall into the cask pit in the unlikely event that it is dropped. As described below, except for the use of a temporary crane, the spent fuel cask crane will be used for the replacement of the existing storage racks in the spent fuel pool (SFP), placement of the temporary rack in the transfer pit, and eventual relocation of racks from the cask pit and transfer pit to the SFP. The spent fuel cask crane is comprised of a main hook rated for 140 tons, as well as an auxiliary hook rated for 20 tons. As described in the DBNPS Updated Safety Analysis Report (USAR) Section 9.1.5, ''Control of Heavy Loads,'' the spent fuel cask crane, including its auxiliary hoist, is subject to compliance with the applicable guidelines of NUREG–0612, ''Control of Heavy Loads at Nuclear Power Plants.'' This will ensure that there will be no significant increase in the probability of a heavy load drop, and that the probability of a heavy load drop will remain extremely small. Due to the limited travel of the spent fuel cask crane, a temporary crane will be used, as necessary, to position existing racks for removal and for final positioning of the new racks. The crane will be designed to meet the intent of NUREG–0612 through a defense-in-depth approach. The temporary crane will only lift the racks several inches above the pool floor, will not be used to lift any heavy loads over fuel assemblies or safety-related equipment, and will not be used to move fuel assemblies. The methods and procedures for handling fuel assemblies during installation activities will not be significantly changed. Based on these considerations, there will be no significant increase in the probability of damage to stored fuel assemblies as a result of installation activities.

For the activities involving the post-installation use of the proposed expanded spent fuel storage capacity, the following previously postulated accident scenarios have been considered: Misloaded or Mislocated Fuel Assembly; Seismic Event; and Fuel Handling Accident. In addition, the effects of a loss of spent fuel pool cooling or level have been evaluated. The probability of the inadvertent misloading or mislocation of a fuel assembly is primarily a function of fuel handling procedures. The probability of a fuel handling accident is primarily a function of fuel handling equipment reliability and fuel handling procedures. The methods and procedures for handling fuel assemblies during normal, post-installation use of the racks will not be significantly changed. In addition, following completion of installation activities, the activities performed in and around the spent fuel pool will not be significantly changed due to the use of the new spent fuel pool racks. The proposed TS changes have no bearing on the probability of a seismic event or the probability of a loss of spent fuel pool cooling or level. Based on these considerations, there will be no significant increase in the probability of an accident previously evaluated as a result of normal, post-installation use of the racks.

1b. Not involve a significant increase in the consequences of an accident previously evaluated because evaluations for each postulated accident have shown that the consequences remain bounded by the consequences from the previously evaluated accidents.

For the installation activities involving the proposed expanded spent fuel storage capacity, heavy load lifts have been given careful consideration. Heavy load lifts are subject to compliance with the applicable guidelines of NUREG–0612. These guidelines include use of defined safe load paths in accordance with approved procedures. This will ensure that there will be no significant increase in the consequences of a heavy load drop, in the unlikely event that one were to occur.

For the activities involving the post-installation use of the proposed expanded spent fuel storage capacity, the following previously postulated accident scenarios have been considered: Misloaded or Mislocated Fuel Assembly; Seismic Event; and Fuel Handling Accident. In addition, the effects of a loss of spent fuel pool cooling or level have been evaluated. The criticality analyses for the new spent fuel pool storage racks require burnup/enrichment limitations similar to those currently in place for the existing racks. These burnup/enrichment limitations are imposed by the proposed changes to TS 3/4.9.13, Refueling Operations-Spent Fuel Assembly Storage. The criticality evaluation for the new racks shows that if an unirradiated fuel assembly of the highest permissible enrichment is placed in an unauthorized storage cell or mislocated outside a storage rack, k_{eff} will be maintained ≤ 0.95, taking credit for soluble boron in the spent fuel pool water. Therefore, there will be no adverse radiological consequences due to the proposed changes.

The results of the seismic evaluation demonstrate that the racks will remain intact and that the structural capability of the pool and liner will not be exceeded. The Auxiliary Building structure will remain intact during a seismic event and will continue to adequately support and protect the fuel racks and pool water inventory, therefore, the rack geometry and cooling to the fuel will be maintained. Thus, there will be no adverse radiological consequences due to the proposed changes.

The new racks do not change the height of the stored fuel relative to any load being handled, and the 72 hour decay time for the fuel assumed in the design basis accident is conservative. Based on this, the design basis fuel handling accident for the pool area remains unchanged.

The mechanical accidents analyses evaluated the extent of rack deformation due to different scenarios. Based on the maximum calculated rack deformation, it was concluded that the criticality and thermal hydraulics limitations were not exceeded. Also, the mechanical accident analyses concluded that the pool liner will not be pierced, and there will be no catastrophic damage to the pool structure. Therefore, the analyzed mechanical accidents will not lead to radiological consequences beyond that already evaluated.

The evaluation of a loss of spent fuel pool cooling shows that sufficient time will be available, before a significant reduction in water level, to restore cooling or to provide a source of makeup water. Therefore, the racks will remain submerged and fuel stored therein will remain sufficiently cooled, and there will be no adverse radiological consequences due to the proposed changes.

The fuel handling area ventilation system will continue to ensure that in the event radioactive material is released from a damaged irradiated fuel assembly, it will be filtered through HEPA and charcoal iodine adsorber filters prior to discharge to the atmosphere. Therefore, the radiological consequences will continue to be mitigated as prior to the proposed changes.

2. Not created the possibility of a new or different kind of accident from any accident previously evaluated because the function and parameters of the components and the associated activities necessary to support safe storage of fuel assemblies in the new racks are similar to those presently in place. The methods and procedures for handling fuel assemblies would not be changed. Therefore, the list of

postulated accidents remains unchanged.

Any event which would modify parameters important to safe fuel storage sufficiently to place them outside of the boundaries analyzed for normal conditions and/or outside of the boundaries previously considered for accidents would be considered a new or different accident. The fuel storage configuration and the existence of the coolant are the parameters that are important to safe fuel storage. The proposed changes do not alter the operating requirements of the plant or of the equipment credited in the mitigation of the design basis accidents, nor do they affect the important parameters required to ensure safe fuel storage. Therefore, the potential for a new or previously unanalyzed accident is not created.

3. Not involve a significant reduction in a margin of safety because for the proposed changes, appropriate evaluations have shown compliance with stipulated safety margins.

The objective of spent fuel storage is to store the fuel assemblies in a subcritical and coolable configuration through all environmental and abnormal loadings, such as a seismic event or a fuel handling accident. The design of the new spent fuel racks meets all applicable requirements for safe fuel storage. The seismic and structural design of the racks preserves the proper margin of safety during normal and abnormal loads. The methodology used in the criticality analysis meets the applicable regulatory guidance. The thermal-hydraulic evaluation demonstrates that the pool will be maintained below the specified thermal limits under the conditions of the maximum heat load and during all credible malfunction scenarios and seismic events. Upon the unlikely event of a complete loss of spent fuel pool cooling, sufficient time will be available, before a significant reduction in water level, to restore cooling or to provide a source of makeup water. Therefore, the racks will remain submerged and fuel stored therein will remain sufficiently cooled. In addition, the results of the fuel handling accident evaluation show that the minimum subcriticality margin will be maintained, cooling will remain adequate, the spent fuel pool structure will not suffer catastrophic damage, and the radiological dose resulting from the release caused by a fuel handling accident will not be increased from that previously considered.

Thus, it is concluded that the proposed changes do not involve a significant reduction in the margin of safety.

Conclusion:

On the basis of the above, the Davis-Besse Nuclear Power Station has determined that the License Amendment Request does not involve a significant hazards considerations. As this License Amendment Request concerns a proposed change to the Technical Specifications that must be reviewed by the Nuclear Regulatory Commission, this License Amendment Request does not constitute an unreviewed safety question.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the **Federal Register** a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, and should cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to Room 6D59, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By October 9th, 2001, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's ''Rules of Practice for Domestic Licensing Proceedings'' in 10 CFR part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, and accessible electronically through the ADAMS Public Electronic Reading Room link at the NRC Web site (http://www.nrc.gov). If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended

petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, and to Mary O'Reilly, FirstEnergy Corporation, 76 South Main Street, Akron, OH., attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

The Commission hereby provides notice that this is a proceeding on an application for a license amendment falling within the scope of section 134 of the Nuclear Waste Policy Act of 1982 (NWPA), 42 U.S.C. 10154. Under section 134 of the NWPA, the Commission, at the request of any party to the proceeding, must use hybrid hearing procedures with respect to ''any matter which the Commission determines to be in controversy among the parties.''

The hybrid procedures in section 134 provide for oral argument on matters in controversy, preceded by discovery under the Commission's rules and the designation, following argument of only those factual issues that involve a genuine and substantial dispute, together with any remaining questions of law, to be resolved in an adjudicatory hearing. Actual adjudicatory hearings are to be held on only those issues found to meet the criteria of section 134 and set for hearing after oral argument.

The Commission's rules implementing section 134 of the NWPA are found in 10 CFR part 2, subpart K, ''Hybrid Hearing Procedures for Expansion of Spent Fuel Storage Capacity at Civilian Nuclear Power Reactors'' (published at 50 FR 41662 dated October 15, 1985). Under those rules, any party to the proceeding may invoke the hybrid hearing procedures by filing with the presiding officer a written request for oral argument under 10 CFR 2.1109. To be timely, the request must be filed within ten (10) days of an order granting a request for hearing or petition to intervene. The presiding officer must grant a timely request for oral argument. The presiding officer may grant an untimely request for oral argument only upon a showing of good cause by the requesting party for the failure to file on time and after providing the other parties an opportunity to respond to the untimely request. If the presiding officer grants a request for oral argument, any hearing held on the application must be conducted in accordance with the hybrid hearing procedures. In essence, those procedures limit the time available for discovery and require that an oral argument be held to determine whether any contentions must be resolved in an adjudicatory hearing. If no party to the proceeding timely requests oral argument, and if all untimely requests for oral argument are denied, then the usual procedures in 10 CFR part 2, subpart G apply.

For further details with respect to this action, see the application for amendment dated December 2, 2000, which is available for public inspection at the Commission's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, and accessible electronically through the ADAMS Public Electronic Reading Room link at the NRC Web site (http://www.nrc.gov).

Dated at Rockville, Maryland, this 30th day of August, 2001.

For the Nuclear Regulatory Commission.

Anthony J. Mendiola,

Chief, Section 2, Project Directorate III, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 01–22412 Filed 9–5–01; 8:45 am]

BILLING CODE 7590–01–P

OFFICE OF PERSONNEL MANAGEMENT

Submission for OMB Review; Comment Request For Reclearance of a Revised Information Collection: RI 98–7

AGENCY: Office of Personnel Management.

ACTION: Notice.

SUMMARY: In accordance with the Paperwork Reduction Act of 1995 (Pub. L. 104–13, May 22, 1995), this notice announces that the Office of Personnel Management (OPM) has submitted to the Office of Management and Budget (OMB) a request for reclearance of a revised information collection. RI 98–7, We Need Important Information About Your Eligibility for Social Security Disability Benefits, is used by OPM to verify receipt of Social Security Administration (SSA) disability benefits, make necessary adjustments to the Federal Employees Retirement