NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice (03–125]

NASA Advisory Committee; Notice of Establishment Pursuant to the Federal Advisory Committee Act, 5 U.S.C. App. Secs. 1 et seq.

AGENCY: National Aeronautics and Space Administration (NASA).

The Administrator of the National Aeronautics and Space Administration has determined that the establishment of an Education Advisory Committee is necessary and in the public interest in connection with the performance of duties imposed upon NASA by law. This determination follows consultation with the Committee Management Secretariat, General Services Administration.

Name of Committee: Education Advisory Committee.

Purpose and Objective: The Committee will advise NASA Administrator on matters related to the Agency's educational program. The Committee will draw on the expertise of its members and other sources to provide its advice and recommendations to the Agency. The Committee will hold meetings and make site visits as necessary to accomplish their responsibilities. The Committee will function solely as an advisory body and will comply fully with the provisions of the Federal Advisory Committee Act.

Balanced Membership Plans: The Committee will consist of non-NASA employees. In addition, there may be associate members selected for Committee Subcommittees or Panels. The Committee may also request appointment of consultants to support specific tasks. Members of the Committee, Subcommittees and Panels will be chosen from among industry, academia, and government with recognized knowledge and expertise in fields relevant to education. Total membership will reflect a balanced view.

Duration: Continuing.

Responsible NASA Official: Dr. Adena Williams Loston, Associate Administrator of the Office of Education, National Aeronautics and Space Administration, 300 E Street, SW., Washington, DC 20546, telephone 202/358–0103.

June W. Edwards,

Advisory Committee Management Officer, National Aeronautics and Space Administration.

[FR Doc. 03–25270 Filed 10–3–03; 8:45 am]

BILLING CODE 7510–01–P

NATIONAL TRANSPORTATION SAFETY BOARD

Sunshine Act Meeting Notice

TIME AND DATE: 9:30 a.m., Tuesday, October 7, 2003.

PLACE: NTSB Board Room, 429 L'Enfant Plaza, SW., Washington, DC 20594.

STATUS: The one item is open to the public.

MATTER TO BE CONSIDERED:

7561 Railroad Accident Report—Collision of Burlington Northern Santa Fe Freight Train with Metrolink Passenger Train at Placentia, California, April 23, 2002.

News Media Contact: Telephone: (202) 314–6000. Individuals requesting specific accommodations should contact Ms. Carolyn Dargan at (202) 314–6305 by Friday, October 3, 2003.

FOR FURTHER INFORMATION CONTACT: Vicky D'Onofrio, (202) 314–6410.

Dated: October 2, 2003.

Vicky D'Onofrio,

Federal Register Liaison Officer.

[FR Doc. 03–25411 Filed 10–2–03; 2:46 pm]

BILLING CODE 7533–01–M

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–302]

Florida Power Corporation, Crystal River Unit 3 Nuclear Generating Plant; Exemption

1.0 Background

Florida Power Corporation (the licensee) is the holder of Facility Operating License No. DPR–72, which authorizes operation of the Crystal River Unit 3 Nuclear Generating Plant (CR–3). The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (the Commission) now or hereafter in effect.

The facility consists of one pressurized-water reactor located in Citrus County, Florida.

2.0 Request/Action

Section 50.44 of Title 10 of the Code of Federal Regulations (10 CFR 50.44), ''Standards for combustible gas control system in light-water-cooled power reactors,'' requires, among other items, that ''[e]ach boiling or pressurized light-water nuclear power reactor fueled with oxide pellets within cylindrical zircaloy or ZIRLO cladding must, as provided in paragraphs (b) through (d) of [that] section, include means for control of hydrogen gas that may be generated, following a postulated loss-of-coolant accident (LOCA) by: (1) [m]etal-water reaction involving the fuel cladding and the reactor coolant, (2) [r]adiolytic decomposition of the reactor coolant, and (3) [c]orrosion of metals.''

Section 50.46 of 10 CFR Part 50, ''Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors,'' requires, among other items, that ''[e]ach boiling or pressurized light-water nuclear power reactor fueled with uranium oxide pellets within cylindrical zircaloy or ZIRLO cladding must be provided with an emergency core cooling system (ECCS) that must be designed so that its calculated cooling performance following postulated [LOCAs] conforms to the criteria set forth in paragraph (b) of [that] section. ECCS cooling performance must be calculated in accordance with an acceptable evaluation model and must be calculated for a number of postulated [LOCAs] of different sizes, locations, and other properties sufficient to provide assurance that the most severe postulated LOCAs are calculated.''

Appendix K to 10 CFR Part 50, ''ECCS Evaluation Models,'' requires, among other items, that the rate of energy release, hydrogen generation, and cladding oxidation from the metal/water reaction shall be calculated using the Baker-Just equation.

Finally, 10 CFR 50.44, 10 CFR 50.46, and 10 CFR part 50, appendix K make no provisions for use of fuel rods clad in a material other than zircaloy or ZIRLO. The licensee has requested the use of Framatome Cogema Fuels (FCF) ''M5'' advanced alloy for fuel rod cladding for the CR–3 operating Cycle 14. The M5 alloy is a proprietary zirconium-based alloy comprised of primarily zirconium (~99 percent) and niobium (~1 percent). The elimination of tin has resulted in superior corrosion resistance and reduced irradiation-induced growth relative to both standard zircaloy (1.7% tin) and low-tin zircaloy (1.2% tin). The addition of niobium increases ductility, which is desirable to avoid brittle failures. Since the chemical composition of the M5 alloy differs from the specifications for zircaloy or ZIRLO, a plant-specific exemption is required to allow the use of the M5 alloy as a cladding material at CR–3.

Section 50.12 of 10 CFR Part 50, ''Specific exemptions,'' states, among other items, that the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of the regulations of this part, which are authorized by law, will not present an

undue risk to the public health and safety, and are consistent with the common defense and security. The Commission will not consider granting an exemption unless special circumstances are present. In accordance with 10 CFR 50.12(a)(2)(ii), special circumstances are present whenever application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule.

3.0 Discussion

The underlying purpose of 10 CFR 50.46 is to ensure that facilities have adequate acceptance criteria for ECCS. On February 4, 2000, the NRC staff approved Topical Report BAW–10227P, ''Evaluation of Advanced Cladding and Structural Material (M5) in PWR Reactor Fuel,'' in which Framatome Cogema Fuels (FCF) demonstrated that the effectiveness of the ECCS will not be affected by a change from zircaloy fuel rod cladding to M5 fuel rod cladding. The analysis described in the topical report also demonstrates that the ECCS acceptance criteria applied to reactors fueled with zircaloy clad fuel are also applicable to reactors fueled with M5 fuel rod cladding.

The underlying purposes of 10 CFR 50.44 and 10 CFR part 50, appendix K, paragraph I.A.5, are to ensure that cladding oxidation and hydrogen generation are appropriately limited during a LOCA and conservatively accounted for in the ECCS evaluation model. Specifically, Appendix K requires that the Baker-Just equation be used in the ECCS evaluation model to determine the rate of energy release, cladding oxidation, and hydrogen generation. In their topical report BAW–10227P, FCF demonstrated that the Baker-Just model is conservative in all post-LOCA scenarios with respect to the use of the M5 advanced alloy as a fuel rod cladding material, and that the amount of hydrogen generated in an M5-clad core during a LOCA will remain within the CR–3 design basis.

The NRC staff has reviewed the licensee's advanced cladding and structural material, M5, for pressurized-water reactor fuel mechanical designs as described in BAW–10227P. In a safety evaluation dated February 4, 2000, for topical report BAW–10227P, the NRC staff concluded that, to the extent and limitations specified in the staff's evaluation, the M5 properties and mechanical design methodology are acceptable for referencing in fuel reload licensing applications. Therefore, since the underlying purposes of 10 CFR 50.44, 10 CFR 50.46, and 10 CFR part 50, appendix K, paragraph I.A.5 are achieved through the use of the M5 advanced alloy as a fuel rod cladding material, the special circumstances required by 10 CFR 50.12(a)(2)(ii) for the granting of exemptions to 10 CFR 50.44 and 10 CFR part 50, appendix K, paragraph I.A.5 exist.

4.0 Conclusion

The Commission has determined that, pursuant to 10 CFR 50.12, this exemption is authorized by law, will not endanger life or property or the common defense and security, and is otherwise in the public interest. Therefore, the Commission hereby grants the licensee an exemption from the requirements of 10 CFR 50.44, 10 CFR 50.46, and 10 CFR part 50, appendix K.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will have no significant impact on the environment (68 FR 55662).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 26th day of September 2003.

For the Nuclear Regulatory Commission.

Ledyard B. Marsh,

Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 03–25243 Filed 10–3–03; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–348 and 50–364]

Southern Nuclear Operating Company, Joseph M. Farley Nuclear Plant; Notice of Receipt and Availability of Application for Renewal of Facility Operating License Nos. NPF–2 and NPF–8 for an Additional 20-Year Period

The U.S. Nuclear Regulatory Commission (NRC) has received an application from Southern Nuclear Operating Company (SNC), on September 15, 2003, filed pursuant to Section 103 of the Atomic Energy Act of 1954, as amended, and 10 CFR part 54 for renewal of Operating License Nos. NPF–2 and NPF–8, which authorize the applicant to operate Joseph M. Farley Nuclear Plant, Units 1 and 2. Farley Nuclear Plant consists of two Westinghouse pressurized water reactor units located about 16.5 miles east of the City of Dothan, in Houston County, Alabama. The operating licenses for Farley Nuclear Plant, Units 1 and 2, expire on June 25, 2017, and March 31, 2021, respectively. The acceptability of the tendered application for docketing and other matters, including an opportunity to request a hearing, will be the subject of subsequent **Federal Register** notices.

Copies of the application are available for public inspection at the Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, or electronically from the Publicly Available Records (PARs) component of the NRC's Agencywide Documents Access and Management System (ADAMS) under Accession Number ML032721356.

The ADAMS Public Electronic Reading Room is accessible from the NRC Web site at *http://www.nrc.gov/NRC/ADAMS/index.html.* In addition, the application is available on the NRC Web page at *http://www.nrc.gov/NRC/REACTOR/LR/index.html.* If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1–800–397–4209, 301–415–4737 or via e-mail to *pdr@nrc.gov.*

The license renewal application for the Farley Nuclear Plant is also available at the Houston Love Memorial Library, 212 West Burdesha Street, Dothan, Alabama.

Dated at Rockville, Maryland, the 30th day of September, 2003.

For the Nuclear Regulatory Commission.

Pao-Tsin Kuo,

Program Director, License Renewal and Environmental Impacts, Division of Regulatory Improvement Programs, Office of Nuclear Reactor Regulation.

[FR Doc. 03–25242 Filed 10–3–03; 8:45 am]

BILLING CODE 7590–01–P

OVERSEAS PRIVATE INVESTMENT CORPORATION

October 15, 2003 Board of Directors Meeting

Time and Date: 4 p.m., Wednesday, October 15, 2003 (Closed to Public).

Place: Offices of the Corporation, Twelfth Floor Board Room, 1100 New York Avenue, NW., Washington, DC

Status: Closed portion will commence at 4 p.m. (approx.).

Matters to be Considered: (Closed to the Public).

1. Discussion of OPIC Product.

2. Insurance Project in Croatia.

FOR FURTHER INFORMATION CONTACT: Information on the meeting may be obtained from Connie M. Downs at (202) 336–8438.