

Document	ADAMS accession No./ Federal Register Citation
Regulatory Guide 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Plants," dated July 2000.	ML003716792.
SECY-11-0089—Options for Proceeding with Future Level 3 Probabilistic Risk Assessment Activities, dated July 7, 2011.	ML11090A039 (Package).

V. Conclusion

For the reasons cited in this document, the NRC is denying PRM-50-121. The current requirements continue to provide reasonable assurance of adequate protection of public health and safety and should not be revised as proposed in the PRM.

Dated: July 24, 2025.

For the Nuclear Regulatory Commission.

Carrie Safford,

Secretary of the Commission.

[FR Doc. 2025-14146 Filed 7-25-25; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

10 CFR Part 50

[Docket No. PRM-50-124; NRC-2022-0178]

Licensing Safety Analysis for Loss-of-Coolant Accidents

AGENCY: Nuclear Regulatory Commission.

ACTION: Petition for rulemaking; denial.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is denying a petition for rulemaking (PRM), dated August 1, 2022, submitted by Ralph O. Meyer. The petition requested that the NRC amend its regulations regarding the licensing safety analysis for loss-of-coolant accidents. The petition was docketed on August 1, 2022, and was assigned Docket No. PRM-50-124. The NRC is denying the petition because the NRC is addressing emergency core cooling system performance with a risk-informed and performance-based regulatory approach rather than a deterministic approach as suggested by the petitioner.

DATES: The docket for the petition for rulemaking, PRM-50-124, is closed on July 28, 2025.

ADDRESSES: Please refer to Docket ID NRC-2022-0178 when contacting the NRC about the availability of information for this action. You may obtain publicly available information related to this action by any of the following methods:

- *Federal Rulemaking Website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2022-0178. Address

questions about NRC dockets to Helen Chang; telephone: 301-415-3228; email: Helen.Chang@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, at 301-415-4737, or by email to PDR.Resource@nrc.gov. For the convenience of the reader, instructions about obtaining materials referenced in this document are provided in the "Availability of Documents" section.

- *NRC's PDR:* The NRC PDR, where you may examine and order copies of publicly available documents, is open by appointment. To make an appointment to visit the PDR, please send an email to PDR.Resource@nrc.gov or call 1-800-397-4209 or 301-415-4737, between 8 a.m. and 4 p.m. eastern time, Monday through Friday, except Federal holidays.

FOR FURTHER INFORMATION CONTACT: Denise Edwards, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone: 301-415-7204, email: Denise.Edwards@nrc.gov.

SUPPLEMENTARY INFORMATION:

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I. The Petition

Section 2.802 of title 10 of the *Code of Federal Regulations* (10 CFR), "Petition for rulemaking," provides an opportunity for any interested person to petition the Commission to issue, amend, or rescind any regulation. On August 1, 2022, the NRC received and docketed a PRM filed by Ralph O. Meyer. On November 23, 2022, the NRC

published a notice of docketing requesting comment on the petition in the **Federal Register** (87 FR 71531). The petition requested that the NRC amend its regulations in 10 CFR 50.46, "Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors." The regulations in 10 CFR 50.46 require the calculation of emergency core cooling system (ECCS) performance following postulated loss-of-coolant accidents (LOCAs) to demonstrate that the acceptance criteria in 10 CFR 50.46(b) are met. The petition requested the elimination of the acceptance criteria for peak cladding temperature (PCT) and maximum cladding oxidation (commonly referred to as maximum local oxidation, or MLO) in 10 CFR 50.46(b). As a replacement for these criteria, the petition requested that the NRC regulations be revised to limit the number of fuel rod ruptures to 1 percent for small-break LOCAs and 10 percent for large-break LOCAs. The petition stated that nuclear power reactors in Germany must show that the proposed fuel rod rupture limits are met.

The NRC identified two issues in the petition, as follows:

Issue 1: The petition asserted that the current acceptance criteria for the calculated PCT and MLO in 10 CFR 50.46(b) do not ensure an easily coolable geometry at moderate and higher fuel burnups due to fuel dispersal.

Issue 2: The petition asserted that the calculations of temperature and oxidation within the burst region of a fuel rod are prone to large uncertainties, and the technical bases for the acceptance criteria on these two parameters are complex and misunderstood. Specifically, the petition stated that the beliefs that the temperature limit prevents autocatalytic oxidation and that a reduction in cladding thickness causes embrittlement are incorrect.

II. Public Comments on the Petition

The NRC invited interested persons to submit comments on the petition (87 FR 71531; November 23, 2022). In response to a request by the Nuclear Energy Institute (NEI), the comment period was extended to March 8, 2023 (88 FR 7012; February 2, 2023). The NRC received 12

comment submissions on the petition. The petitioner submitted eight comment submissions in support of the petition, including submittals after the closure of the public comment period.

Westinghouse Electric Company LLC (Westinghouse), Framatome Inc., the Pressurized Water Reactors Owners Group (PWROG), and NEI each provided a comment submission opposing the petition. A summary of the significant comments and NRC responses is provided below. The NRC evaluation of the petition is provided in Section III, "Evaluation of the Petition."

Several comments on the petition mentioned the then-pending draft final rule commonly referred to as "10 CFR 50.46c." This draft final rule was provided to the Commission in SECY-16-0033, "Draft Final Rule—Performance-Based Emergency Core Cooling System Requirements and Related Fuel Cladding Acceptance Criteria (RIN 3150-AH42)," dated March 16, 2016. On April 11, 2024, the Commission returned the 10 CFR 50.46c draft final rule to the staff in a staff requirements memorandum (SRM) for SECY-16-0033, "Staff Requirements—SECY-16-0033—Draft Final Rule—Performance-Based Emergency Core Cooling System Requirements and Related Fuel Cladding Acceptance Criteria." The NRC comment responses below include responses to the comments on the petition that mentioned the 10 CFR 50.46c draft final rule.

Comment: The petitioner stated that it is appropriate to consider fuel fragmentation, relocation, and dispersal (FFRD) as part of the rulemaking for "Increased Enrichment of Conventional and Accident Tolerant Fuel Designs for Light-Water Reactors" (Increased Enrichment rulemaking).

NRC Response: The NRC agrees with this comment. The Increased Enrichment rulemaking plan was provided to the Commission in SECY-21-0109, "Rulemaking Plan on Use of Increased Enrichment of Conventional and Accident Tolerant Fuel Designs for Light-Water Reactors," dated December 20, 2021. In an SRM dated March 16, 2022, SRM-SECY-21-0109, "Staff Requirements—SECY-21-0109—Rulemaking Plan on Use of Increased Enrichment of Conventional and Accident Tolerant Fuel Designs for Light-Water Reactors," the Commission approved this plan and further directed the staff to consider FFRD issues relevant to higher enrichment and fuel burnup levels as part of the rulemaking.

Comment: Framatome stated that the range of burnups meant by "moderate fuel burnup" in the petition is not clear.

NRC Response: The NRC agrees with this comment. The term "moderate fuel burnup" lacks specificity. The NRC interpreted "moderate fuel burnup" to encompass currently authorized fuel burnups above which FFRD could potentially occur.

Comment: The petitioner stated that licensees should report the number of fuel rod ruptures during a LOCA so the NRC may determine if near-term action is needed to limit the amount of loose material in the core.

NRC Response: The NRC disagrees with this comment. The NRC has reasonable assurance that the extent and consequences of fuel dispersal during a LOCA, for currently authorized fuel burnup levels, would be limited such that core coolability would not be challenged on a large scale. This conclusion is based on research and analyses summarized in SECY-15-0148, "Evaluation of Fuel Fragmentation, Relocation and Dispersal Under Loss-Of-Coolant Accident (LOCA) Conditions Relative to the Draft Final Rule on Emergency Core Cooling System Performance During a LOCA (50.46c)," dated November 30, 2015.

Comment: The petitioner stated that, based on engineering judgment, setting rod rupture limits to 1 percent of the core for a small-break LOCA and 10 percent of the core for a large-break LOCA is reasonable and practical. The petitioner stated that FFRD research should focus on establishing a technical basis for these limits and refine these limits as needed.

NRC Response: The NRC agrees, in part, with this comment. The NRC agrees that research would be needed to establish the technical basis for any fuel rod rupture limits that may be imposed. However, the NRC disagrees with the statement that the petition's proposed limits are reasonable and practical because there currently is not sufficient evidence to support this assertion.

Comment: The petitioner stated that current thermal-hydraulic codes can predict cladding rupture with enough accuracy such that built-in conservatism are not needed to calculate the number of fuel rod bursts.

NRC Response: The NRC agrees, in part, with this comment. The NRC agrees that current thermal-hydraulic codes can predict fuel rod rupture with reasonable accuracy, such that best-estimate plus uncertainty models may be used. However, current LOCA analyses are not focused on fuel rod rupture. The conservatism in current LOCA analyses are necessary to ensure that the PCT and MLO acceptance criteria are met.

Comment: The petitioner stated that, based on analyses and international experience, fuel rod ruptures can be limited to 1 percent for small-break LOCAs and 10 percent for large-break LOCAs. However, this may require some alteration in reactor core design or adjustment of power level to achieve.

NRC Response: The NRC disagrees with this comment. The NRC determined that more research and analysis would be needed to demonstrate that the proposed fuel rod rupture limits could be met, under which operating conditions, and whether such limits are appropriate to ensure a reasonable level of safety.

Comment: The petitioner stated that current LOCA analyses do not address fuel dispersal, and that they assume it does not occur. In reality, fuel dispersal is expected to occur during LOCA.

NRC Response: The NRC agrees with this comment. The NRC acknowledges that fuel dispersal is not accounted for in current LOCA analyses and agrees that fuel dispersal is expected to occur during a LOCA. However, the NRC has reasonable assurance that the extent and consequences of fuel dispersal during a LOCA, for currently authorized fuel burnup levels, would be limited such that core coolability would not be challenged on a large scale.

Comment: The petitioner stated that increased enrichment and burnup could become an issue for long-term core cooling during large-break LOCAs.

NRC Response: The NRC agrees with this comment. The NRC agrees that the higher enrichment and higher burnups that are being considered in the future could increase the magnitude of fuel dispersal and thus it could become challenging to demonstrate long-term cooling. As noted in other comment responses, the Commission has directed the staff to consider FFRD issues relevant to higher enrichment and fuel burnup levels as part of the Increased Enrichment rulemaking.

Comment: The petitioner stated that high burnup fuel is more likely to pulverize than low burnup fuel and is thus a greater threat to long-term core cooling.

NRC Response: The NRC agrees, in part, with this comment. The NRC agrees that high burnup fuel is more likely to pulverize than low burnup fuel, which would very likely result in higher amounts of fuel dispersal. However, the impact of fuel dispersal and particle size on long-term core cooling requires additional research.

Comment: NEI, Westinghouse, Framatome, and the PWROG recommend rejection of the PRM but support rulemaking to address FFRD in

a holistic manner that considers industry initiatives including accident tolerant fuel, increased fuel enrichment, and higher fuel burnups. Several commenters requested that the NRC revise the pending draft final rule for 10 CFR 50.46c to address FFRD issues and industry fuel design initiatives. Framatome indicated that the petition could be considered as part of an update to the draft final rule for 10 CFR 50.46c.

NRC Response: The NRC agrees, in part, with these comments. As discussed in Section IV, “Reasons for Denial,” the NRC plans to address regulatory issues related to accident tolerant fuel, increased fuel enrichment, higher fuel burnups, and FFRD in the Increased Enrichment rulemaking. However, consistent with SRM–SECY–16–0033, the NRC plans to assess the need for further action on the 10 CFR 50.46c rulemaking after the conclusion of the Increased Enrichment rulemaking.

Comment: NEI, Framatome, Westinghouse, and the PWROG stated that revising the ECCS acceptance criteria would require a large amount of effort and resources for industry to comply with them, and this would divert resources from industry’s work on accident tolerant fuel and FFRD research. One commenter noted that criteria similar to the petition’s proposed criteria could be valuable as an alternative or supplemental criteria to demonstrate core coolability, but this would still require significant time and effort to develop and implement.

NRC Response: The NRC agrees, in part, with these comments. Any changes to the ECCS acceptance criteria would require significant effort from the NRC and industry. To the extent that the Increased Enrichment rulemaking would include amendments or alternatives to the ECCS acceptance criteria, the NRC would perform a regulatory analysis to consider the costs and benefits of these approaches. In addition, the cumulative effects of regulations would be considered as part of the rulemaking process to identify and resolve issues that could lead to implementation challenges.

Comment: NEI and the PWROG stated that current ECCS performance criteria in 10 CFR 50.46 remain adequate for the protection of public health and safety. The PWROG noted that the adequacy of the PCT and MLO acceptance criteria was supported by original testing, and subsequent testing to account for changes in plant operations continues to support the basic nature of these acceptance criteria. NEI and the PWROG also noted that industry and NRC analyses performed since 2012 have

confirmed the safe operations of the commercial nuclear fleet.

NRC Response: The NRC agrees with the comments. The current acceptance criteria in 10 CFR 50.46 provide for adequate protection of public health and safety. The NRC agrees that industry and NRC analyses have confirmed the safe operations of the commercial nuclear fleet. As documented in a February 10, 2012, memorandum titled “ECCS Performance Safety Assessment and Audit Report,” the staff completed an ECCS performance safety assessment in 2011 which confirmed, on a plant-specific basis, the safe operation of the commercial nuclear fleet following the identification of concerns with the current acceptance criteria in 10 CFR 50.46. The staff updates this assessment annually to verify the continued safe operation of the fleet. Therefore, with respect to ECCS performance, adequate protection of public health and safety is provided by the current acceptance criteria in 10 CFR 50.46 in conjunction with the annual NRC assessments.

Comment: Westinghouse disagreed with the statement in the petition that the current acceptance criteria become ineffective at moderate fuel burnups because burst fuel rods experience massive fuel loss and do not retain fuel pellets in the fuel rods. The commenter stated that the petition lacks information to support this statement.

NRC Response: The NRC agrees, in part, with the comment. NRC studies to-date predict that 1 to 3 percent of the fuel in the core might be dispersed during a large-break LOCA, using conservative dispersal thresholds, for currently authorized burnup levels. The NRC considers this to be a limited amount of dispersed fuel. The consequences of fuel dispersal at higher burnups are still the topic of ongoing research by the international community.

Comment: Westinghouse and NEI stated that current methods to calculate the MLO and PCT are sufficiently detailed and conservative to demonstrate compliance with 10 CFR 50.46 with a high probability.

NRC Response: The NRC agrees with these comments for currently authorized fuel burnup levels and fuel designs.

Comment: Westinghouse stated that calculations of MLO and PCT using NRC-approved methods support the demonstration of a coolable core geometry and continued safe operation under postulated LOCA conditions.

NRC Response: The NRC agrees with this comment for currently authorized fuel burnup levels and fuel designs. This was documented in SECY–15–0148 and is supported by the NRC staff’s

annual updates to the ECCS performance safety assessment.

Comment: Westinghouse stated that the petition contains a quoted statement from a paper by the German Reactor Safety Commission (RSK) that was taken out of context. The commenter stated that the RSK determined that the criteria in the proposed 10 CFR 50.46c rule are adequate to prevent cladding fragmentation during quench, contrary to what the petition implies.

NRC Response: The NRC agrees with this comment. The petition implied that RSK stated that cladding temperature and oxidation cannot be used in a safety demonstration to comply with regulations. The NRC does not agree with the petition’s assertions about the German regulations. In fact, the NRC confirmed that cladding temperature and oxidation are used to demonstrate compliance with RSK regulations.

Comment: Westinghouse and Framatome stated that the fact that the PCT and MLO criteria in 10 CFR 50.46 are surrogates for the demonstration of the post-quench coolability of the core is well understood and well documented. In addition, the proposed surrogate criteria in 10 CFR 50.46c would maintain the coolability goal and thus maintain plant safety.

NRC Response: The NRC agrees with these comments. Extensive technical bases for 10 CFR 50.46 have been developed and describe the rationale for the surrogate criteria, as summarized by G. Hache and H.M. Chung in “The History of LOCA Embrittlement Criteria,” NUREG/CP–0172, May 2001, pp. 205–237.

Comment: Westinghouse stated that the petition does not provide evidence to support the limits on the percentage of fuel rod bursts proposed in the petition as being appropriate to ensure core coolability. Westinghouse and Framatome stated that the proposed rod burst criteria in the petition do not account for important differences between the various plant designs operating in the United States. These differences could be particularly important when assessing fuel dispersal and its consequences. Framatome stated that using a core-wide burst limit does not make sense given that only high burnup rods are susceptible to fuel dispersal.

NRC Response: The NRC agrees with these comments. The petition does not provide evidence showing that the proposed fuel rod rupture limits would ensure core coolability and the petition does not account for differences in plant designs. As noted in other comment responses, research would be needed to establish the technical basis for any fuel

rod rupture limits that would be imposed.

Comment: NEI, Westinghouse, and Framatome responded to the petition's statement that the proposed fuel rod rupture limits are used in Germany. The German regulatory limits on the percentage of fuel rod ruptures are related to radiological consequences and not used for core coolability assessments. The historical ECCS acceptance criteria for MLO and PCT remain in German regulations.

NRC Response: The NRC agrees with these comments. The NRC does not agree with the petition's assertions about the German regulations.

Comment: Westinghouse, Framatome, and the PWROG stated that, as described in SECY-15-0148, plants continue to operate safely with existing burnup limits and current styles of operation. Any increases above currently licensed fuel burnup levels would require NRC approval. The NRC Research Information Letter 2021-13, "Interpretation of Research on Fuel Fragmentation, Relocation, and Dispersal at High Burnup," dated December 2021, concluded that significant fine fragmentation begins above a fuel burnup threshold of 55 gigawatt days per metric ton uranium, which is conservative.

NRC Response: The NRC agrees with these comments. Today's plants, operating within their burnup limits, are safe.

Comment: Framatome stated that general aspects of FFRD have been known since the 1980s and widespread dispersal that would impact coolability was not expected to be significant.

NRC Response: The NRC agrees, in part, with the comment. The NRC agrees that fuel dispersal would be limited under current licensed burnups and operating practices. However, for higher fuel burnup levels, the NRC recognizes that fuel dispersal could challenge core coolability.

Comment: Westinghouse stated that conclusions from a Westinghouse paper cited in the petition have been superseded by new research and are not appropriate to support the suggested criteria in the petition.

NRC Response: The NRC agrees with this comment. The Westinghouse paper, as explained by the commenter, did not consider fuel pellet thermal conductivity degradation and other material high burnup phenomena.

Comment: One commenter stated that the distribution of ruptured rods in the core is important, and clusters of ruptured rods are more detrimental than more evenly distributed ruptures.

NRC Response: The NRC agrees with this comment based on engineering judgment.

III. Evaluation of the Petition

The petition states that the PCT and MLO acceptance criteria in 10 CFR 50.46(b) would not ensure adequate core cooling during a LOCA because these criteria do not prevent significant fuel dispersal at currently authorized fuel burnup levels and higher.

The PCT and MLO acceptance criteria in 10 CFR 50.46 are important to preventing gross fuel rod failure via a brittle failure mechanism. This ensures that fuel assemblies will remain in a rod-like array within the reactor core that is easier to cool than a rubble pile of fuel and cladding. However, research has shown that the PCT and MLO acceptance criteria do not prevent ductile failure of the cladding (ballooning and rupture), which could lead to fuel dispersal in the reactor core through a rupture in the fuel cladding at higher fuel burnups. Therefore, while the NRC considers the PCT and MLO to be appropriate surrogate metrics for cladding embrittlement, the NRC agrees with the petition's assertion that the acceptance criteria for the calculated PCT and MLO in 10 CFR 50.46(b) do not prevent fuel dispersal due to ductile failures at higher fuel burnup.

For currently authorized fuel burnup levels, the NRC has determined, based on research and analyses, that the extent and consequences of fuel dispersal during a LOCA would be limited such that core coolability would not be challenged on a large scale. However, for higher fuel burnup levels, the NRC recognizes that fuel dispersal could challenge core coolability. The Commission has directed the staff to consider FFRD issues relevant to higher enrichment and fuel burnup levels as part of the Increased Enrichment rulemaking.

The NRC disagrees with the petition's assertion that the technical bases for the PCT and MLO acceptance criteria in 10 CFR 50.46(b) are complex and misunderstood. The calculated PCT and MLO typically occur in the ballooned region of a fuel rod during a LOCA. The NRC acknowledges that the calculation of these parameters is complex and subject to large uncertainties but has found that these calculations can be performed in an appropriately conservative manner. It is well understood by the NRC and experts that perform such calculations that the PCT and MLO acceptance criteria are relatively simple surrogate metrics that correlate with the phenomena leading to

an autocatalytic reaction and to embrittlement, respectively. In addition, the rulemaking record for 10 CFR 50.46 adequately describes the basis for the acceptance criteria.

As a solution to the issues raised in the petition, the petition requested that the NRC regulations be revised to limit the number of fuel rod ruptures to 1 percent for a small-break LOCA and 10 percent for a large-break LOCA. The petition also requested that the PCT and MLO acceptance criteria be eliminated from the NRC regulations. The petition indicates that German nuclear reactors were subject to the fuel rod rupture limits proposed in the petition.

The NRC agrees with several commenters that the petition does not provide an adequate technical basis to support the specific limits on the number of fuel rod ruptures. The German regulations include limits on the PCT and MLO during a LOCA to ensure core coolability that are similar to the acceptance criteria in the NRC regulations. The German regulations also include limits on the number of fuel rod ruptures as indicated in the petition, but these limits are related to the confinement of radioactive materials and not related to core coolability. In addition, the NRC found that the petition does not provide a technical basis to support eliminating the PCT and MLO acceptance criteria from the regulations. Specifically, the petition does not explain how limiting the number of fuel rod ruptures would prevent gross fuel rod failure via a brittle failure mechanism.

IV. Reasons for Denial

The NRC is denying PRM-50-124 because the NRC is addressing emergency core cooling system performance with a risk-informed and performance-based regulatory approach rather than a deterministic approach as suggested by the petitioner. The Commission directed the staff to use a risk-informed and performance-based approach in SRM-SECY-16-0033 and SRM-SECY-21-0109. Under this direction, the staff has begun the Increased Enrichment rulemaking, and the petitioner will have an opportunity to comment on that rulemaking.

V. Availability of Documents

The documents identified in the following table are available to interested persons through one or more of the following methods, as indicated.

Document	ADAMS Accession No./ FEDERAL REG- ISTER Citation
PRM-50-124, Ralph O. Meyer, Petition for Rulemaking, dated August 1, 2022	ML22284A087
PRM-50-124, "Licensing Safety Analysis for Loss-of-Coolant Accidents," notice of docketing and request for comments, dated November 23, 2022.	87 FR 71531
PRM-50-124, "Licensing Safety Analysis for Loss-of-Coolant Accidents," extension of comment period, dated February 2, 2023.	88 FR 7012
Nuclear Energy Institute, Request for Extension of the Comment Period for PRM-50-124, dated January 23, 2023	ML23023A275
Comment (001) from Ralph Meyer on PRM-50-124, dated October 12, 2022	ML23009B712
Comment (002) from Ralph Meyer on PRM-50-124, dated January 12, 2023	ML23031A196
Comment (003) from Zachary Harper of Westinghouse on PRM-50-124, dated February 2, 2023	ML23058A228
Comment (004) from Gayle Elliott on behalf of Framatome Inc., dated February 23, 2023	ML23061A128
Comment (005) from Mike Powell on behalf of Pressurized Water Reactors Owners Group on PRM-50-124, dated March 1, 2023.	ML23062A715
Comment (006) from Frances Pimentel on Behalf of Nuclear Energy Institute on PRM-50-124, dated March 3, 2023	ML23062A716
Comment (007) from Ralph Meyer on PRM-50-124, dated March 14, 2023	ML23074A071
Comment (008) from Ralph Meyer on PRM-50-124, dated July 26, 2023	ML23209A607
Comment (009) from Ralph Meyer on PRM-50-124, dated September 11, 2023	ML23254A398
Comment (010) from Ralph Meyer and Wolfgang Wiesenack on PRM-50-124—Licensing Safety Analysis for Loss-of-Coolant Accidents, dated January 18, 2024.	ML24024A061
Comment (011) from Ralph Meyer on PRM-50-124—Licensing Safety Analysis for Loss-of-Coolant Accidents	ML24100A815
Comment (012) Ralph Meyer on PRM-50-124—Licensing Safety Analysis for Loss-of-Coolant Accidents	ML24239A784
SECY-21-0109, "Rulemaking Plan on Use of Increased Enrichment of Conventional and Accident Tolerant Fuel Designs for Light-Water Reactors," dated December 20, 2021.	ML21232A237
SRM-SECY-21-0109, "Staff Requirements—SECY-21-0109—Rulemaking Plan on Use of Increased Enrichment of Conventional and Accident Tolerant Fuels Designs for Light-Water Reactors," dated March 16, 2022.	ML22075A103
SECY-16-0033, "Draft Final Rule—Performance-Based Emergency Core Cooling System Requirements and Related Fuel Cladding Acceptance Criteria (RIN 3150-AH42)," dated March 16, 2016.	ML15238A947 (Package)
SRM-SECY-16-0033, "Staff Requirements—SECY-16-0033—Draft Final Rule—Performance-Based Emergency Core Cooling System Requirements and Related Fuel Cladding Acceptance Criteria (RIN 3150-AH42).	ML24102A281
SECY-15-0148, "Evaluation of Fuel Fragmentation, Relocation and Dispersal Under Loss-Of-Coolant Accident (LOCA) Conditions Relative to the Draft Final Rule on Emergency Core Cooling System Performance During a LOCA (50.46c)," dated November 30, 2015.	ML15230A200
NRC Research Information Letter 2021-13, "Interpretation of Research on Fuel Fragmentation, Relocation, and Dispersal at High Burnup," dated December 2021.	ML21313A145
NRC Memorandum from Paul M. Clifford to William H. Ruland, "ECCS Performance Safety Assessment and Audit Report," dated February 10, 2012.	ML12041A078
G. Hache and H.M. Chung, "The History of LOCA Embrittlement Criteria," NUREG/CP-0172, May 2001, pp. 205-237	ML011370559

VI. Conclusion

For the reasons cited in this document, the NRC is denying PRM-50-124. The petition did not present any significant new information or arguments that would warrant the requested amendment.

Dated: July 24, 2025.

For the Nuclear Regulatory Commission.

Carrie Safford,

Secretary of the Commission.

[FR Doc. 2025-14215 Filed 7-25-25; 8:45 am]

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FEDERAL DEPOSIT INSURANCE CORPORATION

12 CFR Parts 303, 314, 335, 340, 347, 363, and 380

RIN 3064-AG15

Adjusting and Indexing Certain Regulatory Thresholds

AGENCY: Federal Deposit Insurance Corporation.

ACTION: Notice of proposed rulemaking.

SUMMARY: The Federal Deposit Insurance Corporation (FDIC) is inviting comment on a proposed rule that would amend certain regulatory thresholds in the FDIC's regulations to reflect inflation. Specifically, the proposal would generally update such thresholds to reflect inflation from the date of initial implementation or the most recent adjustment, and provide for future adjustments pursuant to an indexing methodology. The changes set forth in this proposal would provide a more durable regulatory framework by helping to preserve, in real terms, the level of certain thresholds set forth in the FDIC's regulations, thereby avoiding the undesirable and unintended outcome where the scope of applicability for a regulatory requirement changes due solely to inflation rather than actual changes in an institution's size, risk profile or level of complexity.

DATES: Comments must be received on or before September 26, 2025.

ADDRESSES: You may submit comments, identified by RIN 3064-AG15, by any of the following methods:

- **FDIC Website:** <https://www.fdic.gov/federal-register-publications>. Follow instructions for submitting comments on the agency website.
- **Email:** Comments@fdic.gov. Include RIN 3064-AG15 in the subject line of the message.
- **Mail:** Jennifer M. Jones, Deputy Executive Secretary, Attention: Comments—RIN 3064-AG15, Federal Deposit Insurance Corporation, 550 17th Street NW, Washington, DC 20429.
- **Hand Delivery to FDIC:** Comments may be hand-delivered to the guard station at the rear of the 550 17th Street NW building (located on F Street) on business days between 7 a.m. and 5 p.m.
- **Public Inspection:** Comments received, including any personal information provided, may be posted without change to <https://www.fdic.gov/federal-register-publications>.

Commenters should submit only information that the commenter wishes to make available publicly. The FDIC