summarized and/or included in the request for Office of Management and Budget approval of the information collection request; they will also become a matter of public record.

Dated: June 19, 2002.

Gerard F. Fiala,

Office Administrator, Office of Policy and Research.

[FR Doc. 02–16017 Filed 6–24–02; 8:45 am]

BILLING CODE 4510–30–M

NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice (02–076)]

NASA Advisory Council, Minority Business Resource Advisory Committee; Meeting

AGENCY: National Aeronautics and Space Administration (NASA).

ACTION: Notice of meeting.

SUMMARY: In accordance with the Federal Advisory Committee Act, Public Law 92–463, as amended, the National Aeronautics and Space Administration announce a forthcoming meeting of the NASA Advisory Council (NAC), Minority Business Resource Advisory Committee.

DATES: Wednesday, July 31, 2002, 9 a.m. to 4 p.m., and Thursday, August 1, 2002, 9 a.m. to 12 Noon.

ADDRESSES: NASA Headquarters, 300 E. Street, SW., Room 9H40, Washington, DC 20546.

FOR FURTHER INFORMATION CONTACT: Mr. Ralph C. Thomas III, Code K, National Aeronautics and Space Administration, (202) 358–2088.

SUPPLEMENTARY INFORMATION: The meeting will be open to the public up to the seating capacity of the room. The agenda for the meeting is as follows:

—Review of Previous Meeting
—Office of Small and Disadvantaged Business Utilization Update of Activities
—NAC Meeting Report
—Overview of Agency-wide initiatives
—Update of Small Business Program
—Public Comment
—Panel Discussion and Review
—Committee Panel Reports
—Status of Open Committee Recommendations
—New Business

It is imperative that the meeting be held on these dates to accommodate the scheduling priorities of the key participants. Visitors will be requested to sign a visitor's register.

Dated: June 19, 2002.

Sylvia K. Kraemer,

Advisory Committee Management Officer, National Aeronautics and Space Administration.

[FR Doc. 02–15902 Filed 6–24–02; 8:45 am]

BILLING CODE 7510–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–247]

Entergy Nuclear Operations, Inc., Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR–26 issued to Entergy Nuclear Operations, Inc. for operation of the Indian Point Nuclear Generating Unit No. 2 located in Westchester County, New York.

The proposed amendment would revise Technical Specifications (TSs) Section 4.13.A, ''Inspection Requirements,'' to allow the use of the optimum eddy current probe size when performing steam generator tube inspections. The proposed amendment would also correct several grammatical errors.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in Title 10 of the Code of Federal Regulations (10 CFR), Section 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Operation of the facility in accordance with the proposed amendment would not involve a significant increase in the probability of occurrence or consequences of an accident previously evaluated.

The integrity of the steam generator tube portion of the reactor coolant pressure boundary will continue to be monitored, as before, therefore the probability of the failure of the pressure boundary is not affected by the proposed change. Likewise, the probability of the extent of any pressure boundary rupture will not be affected by the proposed changes since the depth and scope of the steam generator tube surveillances are not reduced by the proposed changes. The proposed changes facilitate the application of more advanced diagnostic techniques. The changes involve updating TS Section 4.13.A.3.f. to permit more flexibility in the eddy current inspection probes used in the steam generator tube inspections and to reflect current technological advances in inspection equipment, while still maintaining the minimum 610-mil diameter probe restriction. These changes do not affect possible initiating events for previously evaluated accidents or alter the configuration or operation of the facility. The Limiting Safety System Settings and Safety Limits specified in the current Technical [S]pecifications remain unchanged. Therefore, the proposed changes would not involve a significant increase in the probability or in the consequences of an accident previously evaluated.

2. Operation of the facility in accordance with the proposed amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change is to allow improvements to the inspection techniques and equipment used to perform the steam generator tube inservice inspections. These inspections are only performed while the reactor is in the cold shutdown condition and the steam generators are not capable of affecting the operation of any system, structure or component that maintains the protection of a fission product barrier. The proposed changes facilitate the application of current diagnostic techniques. The safety analysis of the facility remains complete and accurate. There are no physical changes to the facility and the plant conditions for which the design basis accidents have been evaluated are still valid. The operating procedures and emergency procedures are unaffected. Consequently no new failure modes are introduced as a result of the proposed change. Therefore, the proposed change does not create a new accident initiator or precursor, or create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Operation of the facility in accordance with the proposed amendment would not involve a significant reduction in [a] margin of safety.

The integrity of the steam generator portion of the reactor coolant pressure boundary is not affected by the proposed changes to facilitate the application of current diagnostic techniques. Since the elimination of redundant and restrictive controls over the techniques and equipment used to inspect the steam generator tubes does not result in changes to the operation of the facility or the