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FOR FURTHER INFORMATION CONTACT:

Michael Mahoney, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington DC 20555-0001; telephone 301-415-3857, email: Michael.Mahoney@nrc.gov.

SUPPLEMENTARY INFORMATION: The NRC has granted the request of Duke Energy Carolinas, LLC (the licensee) to withdraw its application dated December 15, 2016 (ADAMS Accession No. ML16350A422), for proposed amendments to Renewed Facility Operating License No. NPF-35 and NPF-52 for the Catawba Nuclear Station, Unit Nos. 1 and 2, located in York County, South Carolina.

The submittal dated December 15, 2016, proposed to adopt multiple Technical Specifications Task Force (TSTF) Travelers that would have modified Technical Specification (TS) 3.4.12, "Low Temperature Overpressure Protection (LTOP) System," to increase the time allowed for swapping charging pumps to one hour. The portion related to TSTF-285-A, Revision 1, "Charging Pump Swap LTOP Allowance," is being withdrawn. The licensee's application was noticed previously in the **Federal Register** on April 25, 2017 (82 FR 19099). The licensee provided a request to withdraw the application by letter dated on June 29, 2017 (ADAMS Accession No. ML17180A154).

Dated at Rockville, Maryland, this 19th day of July 2017.

For the Nuclear Regulatory Commission.
Michael Mahoney,

Project Manager, Plant Licensing Branch II-1, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2017-15758 Filed 7-26-17; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 52-025 and 52-026; NRC-2008-0252]

Southern Nuclear Operating Company, Inc., Vogtle Electric Generating Plant, Units 3 and 4; Inspections, Tests, Analyses, and Acceptance Criteria

AGENCY: Nuclear Regulatory Commission.

ACTION: Determination of the successful completion of inspections, tests, and analyses.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) has determined that the inspections, tests, and analyses have been successfully completed, and that the specified acceptance criteria are met for the Vogtle Electric Generating Plant (VEGP), Units 3 and 4.

DATES: The determination of the successful completion of inspections, tests, and analyses for VEGP, Units 3 and 4 is effective July 27, 2017.

ADDRESSES: Please refer to Docket ID NRC-2008-0252 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2008-0252. Address questions about NRC dockets to Carol Gallagher; telephone: 301-415-3463; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-available documents online in the ADAMS Public Documents collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "*Begin Web-based ADAMS Search*." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document.

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FOR FURTHER INFORMATION CONTACT: Chandu Patel, Office of New Reactors, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone: 301-415-3025; email: Chandu.Patel@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Licensee Notification of Completion of ITAAC

Southern Nuclear Operating Company, Inc. (SNC), Georgia Power Company, Oglethorpe Power Corporation, MEAG Power SPVM, LLC., MEAG Power SPVJ, LLC., MEAG Power SPVP, LLC., and the City of Dalton, Georgia, (hereafter called the licensee)

has submitted inspections, tests, analyses, and acceptance criteria (ITAAC) closure notifications (ICNs) under § 52.99(c)(1) of title 10 of the *Code of Federal Regulations* (10 CFR), informing the NRC that the licensee has successfully performed the required inspections, tests, and analyses, and that the acceptance criteria are met for:

VEGP, Unit 3 ITAAC

2.1.01.06.ii (7), 2.1.03.09a.i (81), 2.2.01.06d.i (105), 2.2.03.09b (204), 2.3.05.02.ii (341), 2.5.02.07b (535), and 3.2.00.01c.i (741)

VEGP, Unit 4 ITAAC

2.1.01.06.ii (7), 2.1.03.09a.i (81), 2.2.01.06d.i (105), 2.2.03.09b (204), 2.3.05.02.ii (341), 2.5.02.07b (535), and 3.2.00.01c.i (741)

The ITAAC for VEGP, Unit 3 are in Appendix C of the VEGP, Unit 3 combined license (ADAMS Accession No. ML14100A106). The ITAAC for VEGP, Unit 4 are in Appendix C of VEGP, Unit 4 combined license (ADAMS Accession No. ML14100A135).

II. NRC Staff Determination of Completion of ITAAC

The NRC staff has determined that the specified inspections, tests, and analyses have been successfully completed, and that the specified acceptance criteria are met. The documentation of the NRC staff's determination is in the ITAAC Closure Verification Evaluation Form (VEF) for each ITAAC. The VEF is a form that represents the NRC staff's structured process for reviewing ICNs. Each ICN presents a narrative description of how the ITAAC was completed. The NRC's ICN review process involves a determination on whether, among other things: (1) each ICN provides sufficient information, including a summary of the methodology used to perform the ITAAC, to demonstrate that the inspections, tests, and analyses have been successfully completed; (2) each ICN provides sufficient information to demonstrate that the acceptance criteria of the ITAAC are met; and (3) any NRC inspections for the ITAAC have been completed and any ITAAC findings associated with that ITAAC have been closed.

The NRC staff's determination of the successful completion of these ITAAC is based on information available at this time and is subject to the licensee's ability to maintain the condition that the acceptance criteria are met. If the NRC staff receives new information that suggests the NRC staff's determination on any of these ITAAC is incorrect, then the NRC staff will determine whether to reopen that ITAAC (including

withdrawing the NRC staff's determination on that ITAAC). The NRC staff's determination will be used to support a subsequent finding, pursuant to 10 CFR 52.103(g), at the end of construction that all acceptance criteria in the combined license are met. The ITAAC closure process is not finalized for these ITAAC until the NRC makes an affirmative finding under 10 CFR 52.103(g). Any future updates to the status of these ITAAC will be reflected on the NRC's Web site at <http://www.nrc.gov/reactors/new-reactors/oversight/itaac.html>.

This notice fulfills the NRC staff's obligations under 10 CFR 52.99(e)(1) to publish a notice in the **Federal Register** of the NRC staff's determination of the successful completion of inspections, tests and analyses.

Vogtle Electric Generating Plant Unit 3, Docket No. 5200025

A complete list of the review status for VEGP, Unit 3 ITAAC, including the submission date and ADAMS Accession Number for each ICN received, the ADAMS Accession Number for each VEF, and the ADAMS Accession Numbers for the inspection reports associated with these specific ITAAC, can be found on the NRC's Web site at <http://www.nrc.gov/reactors/new-reactors/new-licensing-files/vog3-icnsr.pdf>.

Vogtle Electric Generating Plant Unit 4, Docket No. 5200026

A complete list of the review status for VEGP Unit 4 ITAAC, including the submission date and ADAMS Accession Number for each ICN received, the ADAMS Accession Number for each VEF, and the ADAMS Accession Numbers for the inspection reports associated with these specific ITAAC, can be found on the NRC's Web site at <http://www.nrc.gov/reactors/new-reactors/new-licensing-files/vog4-icnsr.pdf>.

Dated at Rockville, Maryland, this 18th day of July 2017.

For the Nuclear Regulatory Commission.

Jennifer Dixon-Herrity,

Chief, Licensing Branch 4, Division of New Reactor Licensing, Office of New Reactors.

[FR Doc. 2017-15754 Filed 7-26-17; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-454, 50-455, 50-456, 50-457; NRC-2017-0167]

Exelon Generation Company, LLC; Braidwood Station, Units 1 and 2, and Byron Station, Unit Nos. 1 and 2

AGENCY: Nuclear Regulatory Commission.

ACTION: 10 CFR 2.206 request; receipt.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is giving notice that by petition dated February 8, 2017, Barry Quigley (the petitioner) has requested that the NRC take action with regard to Braidwood Station, Units 1 and 2, and Byron Station, Unit Nos. 1 and 2. The petitioner's requests are included in the **SUPPLEMENTARY INFORMATION** section of this document. **DATES:** July 27, 2017.

ADDRESSES: Please refer to Docket ID NRC-2017-0167 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2017-0167. Address questions about NRC dockets to Carol Gallagher; telephone: 301-415-3463; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-available documents online in the ADAMS Public Documents collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document.

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FOR FURTHER INFORMATION CONTACT: Joel S. Wiebe, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-

0001; telephone: 301-415-6606, email: Joel.Wiebe@nrc.gov.

SUPPLEMENTARY INFORMATION: On February 8, 2017, the petitioner requested that the NRC take action with regard to Braidwood Station, Units 1 and 2, and Byron Station, Unit Nos. 1 and 2 (ADAMS Accession No. ML17061A127). The petitioner requested: (1) Issue a violation under part 50 of title 10 of the *Code of Federal Regulations* (10 CFR), appendix B, Criterion III, Design Control for deficiencies in the analysis of record (AOR) for the main steam isolation valve (MSIV) room pressurization following a high energy line break (HELB), (2) Issue a violation under 10 CFR part 50, appendix B, Criterion XVI, Corrective Action, for failure to update the AOR in a timely manner, (3) Require Exelon (the licensee) to show that the consequences of the secondary missiles resulting from MSIV room pressurization do not have adverse consequences, (4) Issue a demand for information under 10 CFR 2.204, "Demand for information," to compare and contrast the behavior of Exelon management described in the petition with the NRC's policy statement on the attributes of a safety-conscious work environment, and (5) Use Exelon's response to number 4, above, on which to determine whether to issue a "chilling effects" letter.

As the basis for this request, the petitioner states that: (1) Break enthalpies used in the MSIV room pressurization AOR are actually the thermodynamic internal energy of the steam, not the enthalpy. Since in the range of interest, the internal energy is about 13 percent less than the enthalpy, the energy flow to the areas of concern is non-conservative. Steam flow from secondary piping is neglected; (2) Corrective actions to resolve an issue in the AOR are long overdue (8 years) and improperly tracked; (3) A proposed revision to the AOR shows that the MSIV room roof slabs will be ejected by the high pressures in the MSIV rooms becoming potential missiles; and (4) Management dismissed information in the Updated Final Safety Evaluation Report (UFSAR) that supported the concerns about the AOR as "excessive detail" and directed personnel to remove the information. Management dismissed UFSAR internal inconsistency related to the "Break Exclusion Zone" without discussion or review and stated that the information supporting the concern could be deleted as an UFSAR cleanup item. Recently, there was an operability concern where engineering management maintained a