

Effective date: As of the date of issuance, and shall be implemented within 60 days.

Amendment No.: 256.

Renewed Facility Operating License No. NPF-49: Amendment revised the License and Technical Specifications.

Date of initial notice in Federal Register: September 429, 2012 (77 FR 53927).

The supplemental letter contains clarifying information, did not change the scope of the license amendment request, did not change the NRC staff's initial proposed finding of no significant hazards consideration determination, and did not expand the scope of the original **Federal Register** notice.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated January 11, 2013.

No significant hazards consideration comments received: No.

Exelon Generation Company, LLC, and PSEG Nuclear LLC, Docket Nos. 50-277 and 50-278, Peach Bottom Atomic Power Station, Units 2 and 3, York and Lancaster Counties, Pennsylvania

Date of application for amendments: August 29, 2012.

Brief description of amendments: The amendments modified the Technical Specification (TS) requirements for inoperable snubbers by adding Limiting Condition for Operation (LCO) 3.0.8. The amendments also made conforming changes to TS LCO 3.0.1 to reference TS LCO 3.0.8. The proposed changes are based on the Nuclear Regulatory Commission (NRC) approved Technical Specification Task Force (TSTF) standard TS change TSTF-372, Revision 4. A notice of availability for this TS improvement using the consolidated line item improvement process was published by the NRC staff in the **Federal Register** on May 4, 2005 (70 FR 23252).

Date of issuance: January 22, 2013.

Effective date: As of the date of issuance, to be implemented within 60 days.

Amendments Nos.: 285 and 288.

Renewed Facility Operating License Nos. DPR-44 and DPR-56: The amendments revised the License and TSs.

Date of initial notice in Federal Register: October 2, 2012 (77 FR 60150).

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated January 22, 2013.

No significant hazards consideration comments received: No.

Dated at Rockville, Maryland, this 25th day of January 2013.

For the Nuclear Regulatory Commission.

Michele G. Evans,

Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2013-02352 Filed 2-4-13; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards (ACRS)

Meeting of the Joint ACRS Subcommittees on Thermal Hydraulic Phenomena and Materials, Metallurgy and Reactor Fuels; Notice of Meeting

The Joint ACRS Subcommittees on Thermal Hydraulic Phenomena and Materials, Metallurgy and Reactor Fuels will hold a meeting on February 20, 2013, Room T-2B1, 11545 Rockville Pike, Rockville, Maryland.

The meeting will be open to public attendance, with the exception of a portion that may be closed to protect information that is proprietary pursuant to 5 U.S.C. 552(c)(4). The agenda for the subject meeting shall be as follows:

Wednesday, February 20, 2013—8:30 a.m. until 5:00 p.m.

The Subcommittees will review and discuss the thermal conductivity degradation (TCD) issue, how TCD impacts legacy fuel mechanical design codes, and how TCD affects plant safety analysis. The Subcommittees will hear presentations by and hold discussions with the NRC staff and other interested persons regarding this matter.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Weidong Wang (Telephone 301-415-6279 or Email: Weidong.Wang@nrc.gov) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before the meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the DFO with a CD containing each presentation at least thirty minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed

procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 18, 2012, (77 FR 64146-64147).

Detailed meeting agendas and meeting transcripts are available on the NRC Web site at <http://www.nrc.gov/reading-rm/doc-collections/acrs>. Information regarding topics to be discussed, changes to the agenda, whether the meeting has been canceled or rescheduled, and the time allotted to present oral statements can be obtained from the Web site cited above or by contacting the identified DFO. Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with these references if such rescheduling would result in a major inconvenience.

If attending this meeting, please enter through the One White Flint North building, 11555 Rockville Pike, Rockville, MD. After registering with security, please contact Mr. Theron Brown (Telephone 240-888-9835) to be escorted to the meeting room.

Dated: January 30, 2013.

Antonio Dias,

Technical Advisor, Advisory Committee on Reactor Safeguards.

[FR Doc. 2013-02481 Filed 2-4-13; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards (ACRS); Meeting of the ACRS Subcommittee on US-APWR; Notice of Meeting

The ACRS Subcommittee on US-APWR will hold a meeting on February 21-22, 2013, Room T-2B1, 11545 Rockville Pike, Rockville, Maryland.

The meeting will be open to public attendance with the exception of portions that may be closed to protect information that is proprietary pursuant to 5 U.S.C. 552(c)(4). The agenda for the subject meeting shall be as follows:

Thursday, February 21, 2013—8:30 a.m. until 5:00 p.m.; Friday, February 22, 2013—8:30 a.m. until 5:00 p.m.

The Subcommittee will review Chapter 16, "Technical Specifications;" Chapter 17, "Quality Assurance and Reliability Assurance;" and Chapter 19, "Probabilistic Risk Assessment and Severe Accident Evaluation," of the Safety Evaluation Reports (SERs) associated with the US-APWR design certification and the Comanche Peak