

opportunity to request ATAA certification and to provide assistance in filing these requests, as needed.

8. **Action Required.** State administrators should distribute this advisory to appropriate staff. States must adhere to the requirements of federal law that are contained in this advisory.
9. **Inquiries.** States should direct all inquiries to the appropriate ETA Regional Office.

[FR Doc. 05-3420 Filed 2-22-05; 8:45 am]
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NUCLEAR REGULATORY COMMISSION

Agency Information Collection Activities: Submission for the Office of Management and Budget (OMB) Review; Comment Request

AGENCY: U.S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of the OMB review of information collection and solicitation of public comment.

SUMMARY: The NRC has recently submitted to OMB for review the following proposal for the collection of information under the provisions of the Paperwork Reduction Act of 1995 (44 U.S.C. Chapter 35). The NRC hereby informs potential respondents that an agency may not conduct or sponsor, and that a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.

1. *Type of submission, new, revision, or extension:* Extension.

2. *The title of the information collection:*

NUREG/BR-0238, Materials Annual Fee Billing Handbook; NRC Form 628, "Financial EDI Authorization"; NUREG/BR-0254, Payment Methods; NRC Form 629, "Authorization for Payment by Credit Card".

3. *The form number if applicable:*

NRC Form 628, "Financial EDI Authorization"; NRC Form 629, "Authorization for Payment by Credit Card".

4. *How often the collection is required:* Annually.

5. *Who will be required or asked to report:* Anyone doing business with the Nuclear Regulatory Commission including licensees, applicants and individuals who are required to pay a fee for inspections and licenses.

6. *An estimate of the number of annual responses:* 7,330 (10 for NRC

Form 628 and 7,320 for NRC Form 629 and NUREG/BR-0254).

7. *The estimated number of annual respondents:* 7,330 (10 for NRC Form 628 and 7,320 for NRC Form 629 and NUREG/BR-0254).

8. *An estimate of the total number of hours needed annually to complete the requirement or request:* 611 (.8 hour for NRC Form 628 and 610 hours for NRC Form 629 and NUREG/BR-0254).

9. *An indication of whether Section 3507(d), Pub. L. 104-13 applies:* Not applicable.

10. *Abstract:* The U.S. Department of the Treasury encourages the public to pay monies owed the government through use of the Automated Clearinghouse Network and credit cards. These two methods of payment are used by licensees, applicants, and individuals to pay civil penalties, full cost licensing fees, and inspection fees to the NRC.

A copy of the final supporting statement may be viewed free of charge at the NRC Public Document Room, One White Flint North, 11555 Rockville Pike, Room O-1 F21, Rockville, MD 20852. OMB clearance requests are available at the NRC worldwide Web site: <http://www.nrc.gov/public-involve/doc-comment/omb/index.html>. The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions should be directed to the OMB reviewer listed below by March 25, 2005. Comments received after this date will be considered if it is practical to do so, but assurance of consideration cannot be given to comments received after this date. John A. Asalone, Office of Information and Regulatory Affairs (3150-0190), NEOB-10202, Office of Management and Budget, Washington, DC 20503.

Comments can also be e-mailed to John_A._Asalone@omb.eop.gov or submitted by telephone at (202) 395-4650.

The NRC Clearance Officer is Brenda Jo. Shelton, (301) 415-7233.

Dated in Rockville, Maryland, this 14th day of February, 2005.

For the Nuclear Regulatory Commission.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of Information Services.

[FR Doc. 05-3399 Filed 2-22-05; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-413 and 50-414]

Duke Energy Corporation; Concerning the Application for Irradiation of Mixed Oxide Lead Test Assemblies at Catawba Nuclear Station, Units 1 and 2, Supplement No. 1 to Environmental Assessment and Finding of No Significant Impact

The Nuclear Regulatory Commission (NRC) is considering issuance of amendments to the Facility Operating Licenses to permit the use of mixed oxide (MOX) lead test assemblies (LTAs) in one of the two Catawba units and is considering the granting of exemptions from (1) the requirements of Title 10 of the Code of Federal Regulations (10 CFR) Section 50.44(a), 10 CFR 50.46(a)(1) and 10 CFR part 50, Appendix K with respect to the use of M5™ fuel rod cladding; (2) 10 CFR 50.46(a)(1) and Appendix K to part 50 with respect to the use of MOX fuel; and (3) certain physical security requirements of 10 CFR parts 11 and 73 that are usually required at fuel fabrication facilities for the protection of strategic quantities of special nuclear material. A similar request for an exemption from the requirements of 10 CFR 50.44(a) with respect to the use of M5™ fuel rod cladding is not being granted since 10 CFR 50.44 has been changed and an exemption is no longer necessary. The amended licenses and exemptions would apply to Renewed Facility Operating License Nos. NPF-35 and NPF-52, issued to Duke Energy Corporation (Duke, the licensee), for operation of the Catawba Nuclear

Station (Catawba), Units 1 and 2, located in York County, South Carolina.

Pursuant to 10 CFR 51.21, the NRC issued an environmental assessment (EA) and finding of no significant impact (FONSI) on this matter by letter dated August 10, 2004, and also published it in the **Federal Register** on August 17, 2004 (69 FR 51112) (Reference 1). However, in letters dated August 31, September 20, October 29, and December 10, 2004, (References 2, 3, 5 and 6) the licensee stated that certain radiological dose consequence information provided in previous submittals was based on out-of-date input values for design basis accident doses with low enriched uranium (LEU) fuel and provided additional information describing the updated licensing basis dose consequences for the analyzed accidents. Since the EA that was published in the **Federal Register** on August 17, 2004, was based, in part, on the outdated information, the NRC staff is issuing this Supplement to the EA to address the updated information. The dose consequence analyses that were affected by this change are (a) the control room doses for the loss-of-coolant accident analysis (LOCA), the locked rotor analysis (LRA) and the rod ejection analysis (REA), (b) the exclusion area boundary (EAB) doses for the LRA and REA, and (c) the low-population zone (LPZ) doses for the LRA, the REA and the LOCA. Section 5.6, "Design Basis Accident Consequences," is the section of the EA that is affected by this change. This Supplement provides an update of the affected portions of Section 5.6 that supercedes and replaces the comparable portions of Section 5.6 of the EA published on August 17, 2004, to address the information provided in the licensee's letters dated August 31, September 20, October 29, and December 10, 2004, and reaffirms the NRC's conclusions for the EA and the FONSI.

5.6 Design-Basis Accident Consequences (DBAs)

Duke has evaluated the radiological consequences of several categories of postulated DBAs involving MOX LTAs including the category of at-power accidents involving fuel damage to a significant portion of the entire core. These accidents range from the LRA that is calculated to damage 9.5 percent of the fuel assemblies (FAs) in the core (18 FAs) for Unit 1 and 5.0 percent (10 FAs) for Unit 2, the REA that is calculated to damage 50 percent of the core (97 FAs) for either unit, to the large break LOCA that is calculated to damage the full core (193 FAs). Accordingly, considering the

proportion that four MOX LTAs represents of the number of fuel assemblies that are calculated to be damaged by each DBA, the calculated EAB thyroid dose increases that are attributable to the use of MOX are: for the LRA, 14.1 percent for Unit 1 and 25.4 percent for Unit 2; for the REA, 2.62 percent for each unit; and, for the LOCA, 1.32 percent.

The analysis of public doses for the EAB and LPZ resulting from this class of accidents considered by Duke is discussed below. In addition, the NRC staff has evaluated the radiological consequences of affected DBAs on personnel in the control room.

5.6.2 At-Power Accidents

The current licensing basis analyses assume that all FAs (193) are affected by a LOCA. For the LRA, 9.5 percent of the Unit 1 core is assumed to be affected and 5.0 percent of the Unit 2 core is assumed to be affected; for the REA, 50 percent of the core is assumed to be affected. For these events, Duke assumes that the four MOX LTAs are in the affected fuel population displacing four LEU assemblies. Because the dose is directly proportional to the fuel assembly inventory and gap fractions, the impact on the previously analyzed accident doses is based on quantifying the change in fission product release due to replacing up to four LEU fuel assemblies with the MOX LTAs. Although the consequences of these accidents could be determined by updating the current licensing basis analyses, Duke elected to perform a comparative evaluation, which the NRC staff has independently verified.

Duke selected the thyroid dose due to Iodine-131 (I-131) as the evaluation benchmark because the thyroid dose is typically more limiting than the whole body dose in that there is less margin between calculated thyroid doses and its associated dose criterion. Also, I-131 is generally the most significant contributor to thyroid dose due to its abundance and long decay half-life. Duke has determined that the I-131 inventory in a MOX LTA is 9 percent greater than that of an equivalent LEU fuel assembly.

Loss-of-Coolant Accident

For the LOCA, the four MOX LTAs represent 2.1 percent of the 193 assemblies in the core and the potential increase in the iodine release and the thyroid dose would be 1.32 percent. The resulting doses are 90.2 rem at the EAB and 12.9 rem at the LPZ. These doses are below the 300 rem dose reference value of 10 CFR 100.11, "Determination of exclusion area, low population zone,

and population center distance," and are not considered to be significant.

Locked-Rotor Accident

For the LRA in Unit 1, the four MOX LTAs represent 22 percent of the 18 affected assemblies in the core. The potential increase in the iodine release and the thyroid dose is 14.1 percent for Unit 1. The resulting doses are 26.9 rem at the EAB, and 4.6 rem at the LPZ. These doses are below the 300 rem dose reference value of 10 CFR 100.11, and are not considered to be significant.

For the LRA in Unit 2, the four MOX LTAs represent 40 percent of the 10 affected assemblies in the core. The potential increase in the iodine release and the thyroid dose is 25.4 percent for Unit 2. The resulting thyroid doses are 27.8 rem at the EAB, and 4.5 rem at the LPZ. These doses are below the 300 rem dose criterion of 10 CFR 100.11, and are not considered to be significant.

Rod-Ejection Accident

For the REA in Unit 1, the four MOX LTAs represent 4.1 percent of the 97 assemblies in the core assumed to be involved in the postulated accident and the potential increase in the iodine release and the resulting thyroid dose would be 2.62 percent. The resulting calculated thyroid doses are 22.3 rem at the EAB, and 17.8 rem at the LPZ. These doses are below the 300 rem dose criterion of 10 CFR 100.11, and are not considered to be significant.

For the REA in Unit 2, the four MOX LTAs represent 4.1 percent of the 97 assemblies in the core assumed to be involved in the postulated accident and the potential increase in the iodine release and the resulting thyroid dose would be 2.62 percent. Even though the percentage of iodine released from the fuel is the same for Units 1 and 2 (2.62 percent), the release of radioiodine to the environment is greater for Unit 2 due to differences in the design of the steam generators, thus resulting in a higher dose than calculated for Unit 1. The resulting calculated thyroid doses are 31.5 rem at the EAB, and 19.8 rem at the LPZ. These doses are below the 300 rem dose criterion of 10 CFR 100.11, and are not considered to be significant.

5.6.3 Control Room Dose

Control room dose is the only occupational dose that has been previously considered for DBA conditions. The at-power accident with the most severe consequences for the control room personnel is the LOCA; the control room doses from postulated locked-rotor or rod-ejection accidents are bounded by the calculated control

room dose from the LOCA. Duke determined that the resulting control room thyroid dose after a postulated LOCA considering the use of four MOX fuel LTAs would be 13 rem. This is below the NRC staff's 30 rem acceptance criterion and is not considered to be significant.

5.6.4 Conclusion

The DBA with the greatest consequences at the EAB (a LOCA) would result in a calculated offsite dose of 90.2 rem to the thyroid. The DBA with the greatest consequences at the LPZ (a REA) would result in calculated offsite doses of 17.8 and 19.8 rem to the thyroid for Units 1 and 2, respectively. These doses remain below the 300 rem reference value to the thyroid specified in 10 CFR 100.11 for offsite releases. The calculated change in dose consequences at the EAB and at the LPZ that could be attributable to the use of the four MOX fuel LTAs is not significant.

The DBA with the greatest consequences to the control room personnel, a LOCA, would result in a calculated dose of 13 rem to the thyroid. This dose remains below the 30 rem acceptance criterion. The calculated change in dose consequences for control room personnel that could be attributable to the use of the four MOX fuel LTAs is not significant.

The NRC staff concludes that the environmental impact resulting from incremental increases in EAB, LPZ, and control room dose following postulated DBAs that could occur as a result of the irradiation of four MOX LTAs does not represent a significant environmental impact.

11.0 Agencies and Persons Consulted

Related to the publication of the EA in August 2004, (Reference 1), on July 30, 2004, the NRC staff consulted with the South Carolina State official, Mr. Mike Gandy of the Department of Health and Environmental Controls, regarding the environmental impact of the proposed action. The State official had no comments. Related to the issuance of this Supplement to the EA, on February 8, 2005, the NRC staff consulted with the South Carolina State official, Mr. Mike Gandy, of the Department of Health and Environmental Controls, regarding the environmental impact of the proposed action. The State official had no comment.

12.0 References

1. NRC letter to Duke, Catawba Nuclear Station, Units 1 and 2—Environmental Assessment and Finding of No Significant Impact Related to the

Use of Mixed Oxide Lead Test Assemblies (TAC Nos. MB7863, MMB7864, MC0824, MC0825), dated August 10, 2004 (ADAMS ML042230368). Also published in the **Federal Register** on August 17, 2004, 69 FR 51112.

2. Duke letter to NRC, Dose Inputs, August 31, 2004 (ADAMS ML042660144).

3. Duke letter to NRC, Revised Dose Evaluations, September 20, 2004 (ADAMS ML042890343).

4. NRC Letter to Duke, Requesting Additional Information, October 7, 2004 (ADAMS ML042860050).

5. Duke letter to NRC, Response to Request for Additional Information on Revised Dose Evaluations, October 29, 2004 (ADAMS ML043150030).

6. Duke letter to NRC, Additional Information on Revised Dose Evaluations, December 10, 2004 (ADAMS ML043560170).

13.0 Finding of No Significant Impact

On the basis of the EA and Supplement No. 1 to the EA, the NRC reaffirms its conclusion that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated February 27, 2003, and subsequent letters dated September 15, September 23, October 1 (two letters), October 3 (two letters), November 3 and 4, December 10, 2003, and February 2 (two letters), March 1 (three letters), March 9 (two letters), March 16 (two letters), March 26, March 31, April 13, April 16, May 13, June 17, August 31, September 20, October 4, October 29, and December 10, 2004. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209 or (301) 415-4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 14th day of February, 2005.

For the Nuclear Regulatory Commission.

Edwin M. Hackett,

*Project Director, Project Directorate II,
Division of Licensing Project Management,
Office of Nuclear Reactor Regulation.*

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BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-336]

**Dominion Nuclear Connecticut, Inc.;
Millstone Power Station, Unit No. 2;
Exemption**

1.0 Background

Dominion Nuclear Connecticut, Inc. (DNC or the licensee) is the holder of Facility Operating License No. DPR-65, which authorizes operation of the Millstone Power Station, Unit No. 2 (MP2). The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect.

The facility consists of a pressurized water reactor located in New London County, Connecticut.

2.0 Request/Action

By letter dated November 5, 2004, as supplemented by letters dated January 6 and January 25, 2005, the licensee submitted a request for an exemption from the requirements of title 10 of the Code of Federal Regulations (10 CFR) section 50.68(b)(1) for loading, unloading, and handling of the components of the Transnuclear (TN) NUHOMS®-32PT dry cask storage system at MP2.

Section 50.68(b)(1) of 10 CFR sets forth the following requirement that must be met, in lieu of a monitoring system capable of detecting criticality events.

Plant procedures shall prohibit the handling and storage at any one time of more fuel assemblies than have been determined to be safely subcritical under the most adverse moderation conditions feasible by unborated water.

The licensee is unable to satisfy the above requirement for handling the 10 CFR part 72 licensed contents of the TN NUHOMS®-32PT system. Section 50.12(a) allows licensees to apply for an exemption from the requirements of 10 CFR part 50 if the regulation is not necessary to achieve the underlying purpose of the rule and other conditions are met. The licensee stated in the application that compliance with 10 CFR 50.68(b)(1) is not necessary for