Estimated Annual Burden Per Voluntary Respondent: 30 hours.

Estimated Total Annual Burden on Respondents: 150 hours.

Jacqueline Agtuca,

Chief of Staff.

[FR Doc. 01–10917 Filed 5–1–01; 8:45 am]

BILLING CODE 7565–01–P

NORTHEAST DAIRY COMPACT COMMISSION

Notice of Meeting

AGENCY: Northeast Dairy Compact Commission.

ACTION: Notice of meeting.

SUMMARY: The Compact Commission will hold its regular monthly meeting to consider matters relating to administration and enforcement of the price regulation. This meeting will be held in Rhode Island, continuing the Commission's program of holding a meeting in each of the Compact states. In addition to receiving reports and recommendations of its standing Committees, the Commission will receive a number of informational reports, including reports on the operation of the wholesale and retail markets and about the impact of the price regulation on the Rhode Island WIC Program.

DATES: The meeting will begin at 10 a.m. on Friday, May 11, 2001.

ADDRESSES: The meeting will be held at the Newport Marriott Hotel, 25 America's Cup Avenue, Newport, Rhode Island.

FOR FURTHER INFORMATION CONTACT: Daniel Smith, Executive Director, Northeast Dairy Compact Commission, 64 Main Street, Room 21, Montpelier, VT 05602. Telephone (802) 229–1941.

Authority: 7 U.S.C. 7256.

Dated: April 25, 2001.

Daniel Smith,

Executive Director.

[FR Doc. 01–10888 Filed 5–1–01; 8:45 am]

BILLING CODE 1650–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–338 and 50–339]

Virginia Electric and Power Company, North Anna Power Station, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from the requirements of Title 10 of the Code of Federal Regulations (10 CFR) part 50, appendix G, for Facility Operating License Nos. NPF–4 and NPF–7, issued to Virginia Electric and Power Company (the licensee), for operation of the North Anna Power Station, Units 1 and 2, located in Louisa County, Virginia.

Environmental Assessment

Identification of the Proposed Action

10 CFR Part 50, Appendix G, requires that the pressure-temperature (P–T) limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic or leak testing conditions. Specifically, 10 CFR part 50, Appendix G, states that ''[t]he appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions.'' Appendix G of 10 CFR part 50 specifies that the requirements for these limits are contained in the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Appendix G.

To address provisions of an amendment to the Technical Specifications P–T limits and low-temperature overpressure protection (LTOP) system setpoints, the licensee requested in its submittal dated June 22, 2000, as supplemented on January 4, February 14, March 13, and March 22, 2001, that the NRC staff exempt North Anna Power Station from the requirements of 10 CFR Part 50, Appendix G, to allow the use of ASME Code Case N–641.

Code Case N–641 permits the use of an alternate reference fracture toughness (K_{IC} fracture toughness curve instead of the K_{IA} fracture toughness curve) for reactor vessel materials in determining the P–T limits, LTOP system setpoints and T_{enable}, and provides for plant-specific evaluation of T_{enable}. Since the K_{IC} fracture toughness curve shown in ASME Section XI, Appendix A, Figure A–2200–1 (the K_{IC} fracture toughness curve) provides greater allowable fracture toughness than the corresponding K_{Ia} fracture toughness curve of ASME Section XI, Appendix G, Figure G–2210–1 (the K_{Ia} fracture toughness curve), and a plant-specific evaluation of T_{enable} would give lower values of T_{enable} than use of a generic bounding evaluation for T_{enable}, use of Code Case N–641 for establishing the P–T limits, LTOP system setpoints and T_{enable} would be less conservative than the methodology currently endorsed by 10 CFR Part 50, Appendix G. Although the use of the K_{IC} fracture toughness curve in ASME Code Case N–641 was recently incorporated into Appendix G to Section XI of the ASME Code, an exemption is still needed because 10 CFR Part 50, Appendix G requires a licensee's analysis to use an edition and addenda of Section XI of the ASME Code incorporated by reference into 10 CFR Part 50, section 50.55a, i.e., the editions through 1995 and addenda through the 1996 addenda (which do not include the provisions of Code Case N–641). Therefore, an exemption to apply the Code case is required by 10 CFR Part 50, section 50.60. The proposed action is in accordance with the licensee's application for exemption dated June 22, 2000, as supplemented by letters dated January 4, February 14, March 13, and March 22, 2001.

The Need for the Proposed Action

ASME Code Case N–641 is needed to revise the method used to determine the reactor coolant system (RCS) P–T limits, LTOP setpoints, and T_{enable}.

The purpose of 10 CFR part 50, Section 50.60(a), and 10 CFR part 50, appendix G, is to protect the integrity of the reactor coolant pressure boundary in nuclear power plants. This is accomplished through these regulations that, in part, specify fracture toughness requirements for ferritic materials of the reactor coolant pressure boundary. Pursuant to 10 CFR part 50, appendix G, it is required that P–T limits for the RCS be at least as conservative as those obtained by applying the methodology of the ASME Code, Section XI, Appendix G.

Current overpressure protection system (OPPS) setpoints produce operational constraints by limiting the P–T range available to the operator to heat up or cool down the plant. The operating window through which the operator heats up and cools down the RCS becomes more restrictive with continued reactor vessel service. Reducing this operating window could potentially have an adverse safety impact by increasing the possibility of inadvertent OPPS actuation due to pressure surges associated with normal plant evolutions such as reactor coolant pump start and swapping operating charging pumps with the RCS in a water-solid condition. The impact on the P–T limits and OPPS setpoints has been evaluated for an increased service period for operation to 32.3 effective full-power years (EFPYs) for Unit 1 and 34.3 EFPYs for Unit 2, based on ASME Code, Section XI, Appendix G requirements. The results indicate that these OPPS setpoints would significantly restrict the ability to perform plant heatup and cooldown,

create an unnecessary burden to plant operations, and challenge control of plant evolutions required with OPPS enabled. Continued operation of North Anna Units 1 and 2 with P–T curves developed to satisfy ASME Code, Section XI, Appendix G, requirements without the relief provided by ASME Code Case N–641 would unnecessarily restrict the P–T operating window, especially at low temperature conditions.

Use of the K_{Ic} curve in determining the lower bound fracture toughness of RPV steels is more technically correct than use of the K_{Ia} curve since the rate of loading during a heatup or cooldown is slow and is more representative of a static condition than a dynamic condition. The K_{Ic} curve appropriately implements the use of static initiation fracture toughness behavior to evaluate the controlled heatup and cooldown process of a reactor vessel. The staff has required use of the conservatism of the K_{Ia} curve since 1974, when the curve was adopted by the ASME Code. This conservatism was initially necessary due to the limited knowledge of the fracture toughness of RPV materials at that time. Since 1974, additional knowledge has been gained about RPV materials, which demonstrates that the lower bound on fracture toughness provided by the K_{Ia} curve greatly exceeds the margin of safety required, and that the K_{IC} curve is sufficiently conservative, to protect the public health and safety from potential RPV failure. Application of ASME Code Case N–641 will provide results that are sufficiently conservative to ensure the integrity of the reactor coolant pressure boundary while providing P–T curves that are not overly restrictive. Implementation of the proposed P–T curves, as allowed by ASME Code Case N–641, does not significantly reduce the margin of safety.

In the associated exemption, the NRC staff has determined that, pursuant to 10 CFR part 50, section 50.12(a)(2)(ii), the underlying purpose of the regulation will continue to be served by the implementation of ASME Code Case N–641.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that the proposed action provides adequate margin of safety against brittle failure of the reactor coolant pressure boundary. The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not involve any historic sites. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (*i.e.,* the ''no-action'' alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the North Anna Power Station, Units 1 and 2, dated April 1973.

Agencies and Persons Consulted

In accordance with its stated policy, on April 2, 2001, the staff consulted with the Virginia State official, Mr. J. Dekrafft of the Radiological Health Program of the Virginia Department of Health, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Signficant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated June 22, 2000, as supplemented by letters dated January 4, February 14, March 13, and March 22, 2001. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the ADAMS Public Library component on the NRC Web site, *http://www.nrc.gov* (the Electronic Reading Room).

Dated at Rockville, Maryland, this 26th day of April 2001.

For the Nuclear Regulatory Commission.

Gordon E. Edison,

Senior Project Manager, Section 1, Project Directorate II, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 01–10965 Filed 5–1–01; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–482]

Wolf Creek Nuclear Operating Corporation, Wolf Creek Generating Station; Notice of Consideration of Approval of Application Regarding Proposed Corporate Restructuring of Kansas City Power & Light Company and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an order under 10 CFR 50.80 approving the indirect transfer of Facility Operating License No. NPF–42 for Wolf Creek Generating Station (WCGS) as held by Kansas City Power & Light Company (KCPL), one of three joint owners of WCGS, and Wolf Creek Nuclear Operating Corporation, the operator of the facility, to a new holding company for KCPL, to the extent such indirect transfer would occur in connection with a proposed restructuring of KCPL. The facility is located in Coffey County, Kansas.

According to the February 20, 2001, application filed by KCPL, which was supplemented by letters dated February 27, March 5, and March 8, 2001, from counsel for KCPL, the proposed restructuring of KCPL encompasses the formation of a newly formed holding company as yet unnamed (''*HoldingCo*''). Upon the proposed restructuring, KCPL will cease to be publicly-traded and become a wholly-owned subsidiary of *HoldingCo,* but it will retain ownership of its regulated electric power generation, transmission, and distribution assets, including its interests in WCGS and Wolf Creek Nuclear Operating Corporation (WCNOC). No direct transfer of the license as now held by KCPL and WCNOC to *HoldingCo* is being proposed.

WCNOC would remain as the managing agent for the joint owner licensees (KCPL, Kansas Gas and Electric Company, and Kansas Electric