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NUCLEAR REGULATORY COMMISSION

10 CFR Part 72

RIN 3150–AG97

List of Approved Spent Fuel Storage Casks: HI-STORM 100 Revision

AGENCY: Nuclear Regulatory Commission.

ACTION: Direct final rule.

SUMMARY: The Nuclear Regulatory Commission (NRC) is amending its regulations revising the Holtec International HI-STORM 100 cask system listing within the list of approved spent fuel storage casks to include Amendment 1 to Certificate of Compliance (CoC) Number 1014. This amendment will modify the present cask system design, under a general license, to: add four new multipurpose canisters; add new containers for damaged fuel; add the HI-STORM 100S overpack and the 100A and 100SA high-seismic anchored overpacks; allow the storage of high-burnup fuel; delete the Technical Specifications for special requirements for the first systems in place and for training requirements and relocate these requirements to the main body of CoC 1014; and allow the storage of selected nonfuel hardware. The amendment will also use revised thermal analysis tools to include natural convection heat transfer, revise the helium backfill requirements to allow a helium density measurement to be used, allow a helium drying system rather than the existing vacuum drying system, and require soluble boron during canister loading for certain higher enriched fuels. In addition, modifications will be made to applicable CoC conditions and sections of Appendix A and Appendix B to the CoC to reflect the changes.

DATES: The final rule is effective June 10, 2002, unless significant adverse comments are received by April 26, 2002. A significant adverse comment is a comment where the commenter explains why the rule would be inappropriate, including challenges to the rule's underlying premise or approach, or would be ineffective or unacceptable without a change. If the rule is withdrawn, timely notice will be published in the **Federal Register**.

ADDRESSES: Submit comments to: Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, Attn: Rulemakings and Adjudications Staff. Deliver comments to 11555 Rockville Pike, Rockville, MD, between 7:30 a.m. and 4:15 p.m. on Federal workdays.

Certain documents related to this rulemaking, as well as all public comments received on this rulemaking, may be viewed and downloaded electronically via the NRC's rulemaking website at *http://ruleforum.llnl.gov.* You may also provide comments via this website by uploading comments as files (any format) if your web browser supports that function. For information about the interactive rulemaking site, contact Ms. Carol Gallagher, (301) 415–5905; e-mail CAG@nrc.gov.

Certain documents related to this rule, including comments received by the NRC, may be examined at the NRC Public Document Room, 11555 Rockville Pike, Rockville, MD. For more information, contact the NRC Public Document Room (PDR) Reference staff at 1–800–397–4209, 301–415–4737 or by e-mail to *pdr@nrc.gov.*

Documents created or received at the NRC after November 1, 1999, are also available electronically at the NRC's Public Electronic Reading Room on the Internet at *http://www.nrc.gov/reading-rm.html.* From this site, the public can gain entry into the NRC's Agencywide Documents Access and Management System (ADAMS), which provides text and image files of NRC's public documents. An electronic copy of the proposed CoC and preliminary safety evaluation report (SER) can be found under ADAMS Accession No. ML013330457. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC PDR Reference staff at 1–800–397–4209, 301–415–4737 or by e-mail to *pdr@nrc.gov.*

CoC No. 1014, the revised Technical Specifications, the underlying Safety Evaluation Report for Amendment No. 1, and the Environmental Assessment, are available for inspection at the NRC Public Document Room, 11555 Rockville Pike, Rockville, MD. Single copies of these documents may be obtained from Jayne M. McCausland, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, telephone (301) 415–6219, e-mail *jmm2@nrc.gov.*

FOR FURTHER INFORMATION CONTACT: Jayne M. McCausland, telephone (301) 415–6219, e-mail *jmm2@nrc.gov,* of the Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

SUPPLEMENTARY INFORMATION:

Background

Section 218(a) of the Nuclear Waste Policy Act of 1982, as amended (NWPA), requires that ''[t]he Secretary [of the Department of Energy (DOE)] shall establish a demonstration program, in cooperation with the private sector, for the dry storage of spent nuclear fuel at civilian nuclear power reactor sites, with the objective of establishing one or more technologies that the [Nuclear Regulatory] Commission may, by rule, approve for use at the sites of civilian nuclear power reactors without, to the maximum extent practicable, the need for additional site-specific approvals by the Commission.'' Section 133 of the NWPA states, in part, that ''[t]he Commission shall, by rule, establish procedures for the licensing of any technology approved by the Commission under Section 218(a) for use at the site of any civilian nuclear power reactor.''

To implement this mandate, the NRC approved dry storage of spent nuclear fuel in NRC-approved casks under a general license by publishing a final rule in 10 CFR Part 72 entitled ''General License for Storage of Spent Fuel at Power Reactor Sites'' (55 FR 29181; July 18, 1990). This rule also established a new Subpart L within 10 CFR Part 72, entitled ''Approval of Spent Fuel Storage Casks'' containing procedures and criteria for obtaining NRC approval of spent fuel storage cask designs. The NRC subsequently issued a final rule on May 1, 2000 (65 FR 25241), that approved the Holtec HI-STORM 100 cask design and added it to the list of

NRC-approved cask designs in § 72.214 as Certificate of Compliance Number (CoC No.) 1014.

Discussion

On July 3, 2001, and as supplemented on August 13 and 17, and October 5, 12, and 19, 2001, the certificate holder, Holtec International, submitted an application to the NRC to amend CoC No. 1014 to permit a Part 72 licensee to: (1) Add four new multipurpose canisters—three for pressurized water reactor fuel and one for boiling water reactor fuel; (2) add new containers for damaged fuel; (3) add the HI-STORM 100S overpack and the 100A and 100SA high-seismic anchored overpacks; (4) allow the storage of high-burnup fuel; (5) delete the Technical Specifications for special requirements for the first systems in place and for training requirements and relocate these requirements to the main body of CoC 1014; and (6) allow the storage of selected nonfuel hardware. The amendment also will utilize revised thermal analysis tools to include natural convection heat transfer, revise the helium backfill requirements to allow a helium density measurement to be used, allow a helium drying system rather than the existing vacuum drying system, and require soluble boron during canister loading for certain higher enriched fuels. In addition, modifications will be made to Conditions 1.a., 1.b., 2, 3, 5, 9, 10, and 11 of the CoC; Sections 3.0 and 5.0 of Appendix A to the CoC; and Sections 1.0, 2.0, and 3.0 of Appendix B to the CoC to reflect the changes. No other changes to the Holtec HI-STORM 100 cask system design were requested in this application. The NRC staff performed a detailed safety evaluation of the proposed CoC amendment request and found that an acceptable safety margin is maintained. In addition, the NRC staff has determined that there is still reasonable assurance that public health and safety and the environment will be adequately protected.

This direct final rule revises the Holtec HI-STORM 100 cask design listing in § 72.214 by adding Amendment No. 1 to CoC No. 1014. The amendment consists of changes to the Technical Specifications as described above in the ''Discussion'' portion of this document. The particular Technical Specifications that are changed are identified in the NRC staff's Safety Evaluation Report for Amendment 1.

The amended Holtec HI-STORM 100 cask system, when used in accordance with the conditions specified in the CoC, the Technical Specifications, and NRC regulations, will meet the requirements of Part 72; thus, adequate protection of public health and safety will continue to be ensured.

Discussion of Amendments by Section

Section 72.214 List of Approved Spent Fuel Storage Casks

Certificate No. 1014 is revised by adding the effective date of the initial certificate, and the effective date of Amendment Number 1.

Procedural Background

This rule is limited to the changes contained in Amendment 1 to CoC No. 1014 and does not include other aspects of the Holtec HI-STORM 100 cask system design. The NRC is using the ''direct final rule procedure'' to issue this amendment because it represents a limited and routine change to an existing CoC that is expected to be noncontroversial. Adequate protection of public health and safety continues to be ensured. The amendment to the rule will become effective on June 10, 2002. However, if the NRC receives significant adverse comments by April 26, 2002, then the NRC will publish a document that withdraws this action and will address the comments received in response to the proposed amendments published elsewhere in this issue of the **Federal Register**. A significant adverse comment is a comment where the commenter explains why the rule would be inappropriate, including challenges to the rule's underlying premise or approach, or would be ineffective or unacceptable without a change. A comment is adverse and significant if:

(1) The comment opposes the rule and provides a reason sufficient to require a substantive response in a notice-and-comment process. For example, in a substantive response:

(a) The comment causes the NRC staff to reevaluate (or reconsider) its position or conduct additional analysis;

(b) The comment raises an issue serious enough to warrant a substantive response to clarify or complete the record; or

(c) The comment raises a relevant issue that was not previously addressed or considered by the NRC staff.

(2) The comment proposes a change or an addition to the rule, and it is apparent that the rule would be ineffective or unacceptable without incorporation of the change or addition.

(3) The comment causes the NRC staff to make a change (other than editorial) to the CoC or TS.

These comments will be addressed in a subsequent final rule. The NRC will not initiate a second comment period on this action. However, if the NRC receives significant adverse comments by April 26 2002, then the NRC will publish a document that withdraws this action and will address the comments received in response to the proposed amendments published elsewhere in this issue of the **Federal Register**.

Agreement State Compatibility

Under the ''Policy Statement on Adequacy and Compatibility of Agreement State Programs'' approved by the Commission on June 30, 1997, and published in the **Federal Register** on September 3, 1997 (62 FR 46517), this rule is classified as compatibility Category ''NRC.'' Compatibility is not required for Category ''NRC'' regulations. The NRC program elements in this category are those that relate directly to areas of regulation reserved to the NRC by the Atomic Energy Act of 1954, as amended (AEA) or the provisions of the Title 10 of the Code of Federal Regulations. Although an Agreement State may not adopt program elements reserved to NRC, it may wish to inform its licensees of certain requirements via a mechanism that is consistent with the particular State's administrative procedure laws, but does not confer regulatory authority on the State.

Plain Language

The Presidential Memorandum dated June 1, 1998, entitled ''Plain Language in Government Writing'' directed that the Government's writing be in plain language. The NRC requests comments on this direct final rule specifically with respect to the clarity and effectiveness of the language used. Comments should be sent to the address listed under the heading **ADDRESSES** above.

Voluntary Consensus Standards

The National Technology Transfer Act of 1995 (Pub. L. 104–113) requires that Federal agencies use technical standards that are developed or adopted by voluntary consensus standards bodies unless the use of such a standard is inconsistent with applicable law or otherwise impractical. In this direct final rule, the NRC would revise the Holtec HI-STORM 100 cask system design listed in § 72.214 (List of NRC-approved spent fuel storage cask designs). This action does not constitute the establishment of a standard that establishes generally applicable requirements.

Finding of No Significant Environmental Impact: Availability

Under the National Environmental Policy Act of 1969, as amended, and the NRC regulations in Subpart A of 10 CFR

Part 51, the NRC has determined that this rule, if adopted, would not be a major Federal action significantly affecting the quality of the human environment and, therefore, an environmental impact statement is not required. The rule would amend the CoC for the Holtec HI-STORM 100 cask system within the list of approved spent fuel storage casks that power reactor licensees can use to store spent fuel at reactor sites under a general license. The amendment will modify the present cask system design to: (1) Add four new multipurpose canisters—three for pressurized water reactor fuel and one for boiling water reactor fuel; (2) add new containers for damaged fuel; (3) add the HI-STORM 100S overpack and the 100A and 100SA high-seismic anchored overpacks; (4) allow the storage of high-burnup fuel; (5) delete the Technical Specification for special requirements for the first systems in place and for training requirements and relocate these requirements to the main body of CoC 1014; and (6) allow the storage of selected nonfuel hardware. The amendment also will utilize revised thermal analysis tools to include natural convection heat transfer, revise the helium backfill requirements to allow a helium density measurement to be used, allow a helium drying system rather than the existing vacuum drying system, and require soluble boron during canister loading for certain higher enriched fuels. In addition, modifications will be made to Conditions 1.a., 1.b., 2, 3, 5, 9, 10, and 11 of the CoC; Sections 3.0 and 5.0 of Appendix A to the CoC; and Sections 1.0, 2.0, and 3.0 of Appendix B to the CoC to reflect the changes. The environmental assessment and finding of no significant impact on which this determination is based are available for inspection at the NRC Public Document Room, 11555 Rockville Pike, Rockville, MD. Single copies of the environmental assessment and finding of no significant impact are available from Jayne M. McCausland, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, telephone (301) 415–6219, email *jmm2@nrc.gov.*

Paperwork Reduction Act Statement

This direct final rule does not contain a new or amended information collection requirement subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 *et seq.*). Existing requirements were approved by the Office of Management and Budget, Approval Number 3150–0132.

Public Protection Notification

If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

Regulatory Analysis

On July 18, 1990 (55 FR 29181), the NRC issued an amendment to 10 CFR Part 72 to provide for the storage of spent nuclear fuel under a general license in cask designs approved by the NRC. Any nuclear power reactor licensee can use NRC-approved cask designs to store spent nuclear fuel if it notifies the NRC in advance, spent fuel is stored under the conditions specified in the cask's CoC, and the conditions of the general license are met. A list of NRC-approved cask designs is contained in § 72.214. On May 1, 2000 (65 FR 25241), the NRC issued an amendment to Part 72 that approved the Holtec HI-STORM 100 cask design by adding it to the list of NRC-approved cask designs in § 72.214. On July 3, 2001, and as supplemented on August 13 and 17, and October 5, 12, and 19, 2001, the certificate holder, Holtec International, submitted an application to the NRC to amend CoC No. 1014 to permit a Part 72 licensee to: (1) Add four new multipurpose canisters—three for pressurized water reactor fuel and one for boiling water reactor fuel; (2) add new containers for damaged fuel; (3) add the HI-STORM 100S overpack and the 100A and 100SA high-seismic anchored overpacks; (4) allow the storage of high-burnup fuel; (5) delete the Technical Specifications for special requirements for the first systems in place and for training requirements and relocate these requirements to the main body of CoC 1014; and (6) allow the storage of selected nonfuel hardware. The amendment also will utilize revised thermal analysis tools to include natural convection heat transfer, revise the helium backfill requirements to allow a helium density measurement to be used, allow a helium drying system rather than the existing vacuum drying system, and require soluble boron during canister loading for certain higher enriched fuels. In addition, modifications will be made to Conditions 1.a., 1.b., 2, 3, 5, 9, 10, and 11 of the CoC; Sections 3.0 and 5.0 of Appendix A to the CoC; and Sections 1.0, 2.0, and 3.0 of Appendix B to the CoC to reflect the changes.

The alternative to this action is to withhold approval of this amended cask system design and issue a site-specific license to each licensee. This alternative would cost both the NRC and the utilities more time and money because each utility would have to pursue an exemption or a site-specific license.

Approval of the direct final rule will eliminate this problem and is consistent with previous NRC actions. Further, the direct final rule will have no adverse effect on public health and safety. This direct final rule has no significant identifiable impact or benefit on other Government agencies. Based on this discussion of the benefits and impacts of the alternatives, the NRC concludes that the requirements of the direct final rule are commensurate with the NRC's responsibilities for public health and safety and the common defense and security. No other available alternative is believed to be as satisfactory, and thus, this action is recommended.

Regulatory Flexibility Certification

In accordance with the Regulatory Flexibility Act of 1980 (5 U.S.C. 605(b)), the NRC certifies that this rule will not, if issued, have a significant economic impact on a substantial number of small entities. This direct final rule affects only the licensing and operation of nuclear power plants, independent spent fuel storage facilities, and Holtec International. The companies that own these plants do not fall within the scope of the definition of ''small entities'' set forth in the Regulatory Flexibility Act or the Small Business Size Standards set out in regulations issued by the Small Business Administration at 13 CFR Part 121.

Backfit Analysis

The NRC has determined that the backfit rule (10 CFR 50.109 or 10 CFR 72.62) does not apply to this direct final rule because this amendment does not involve any provisions that would impose backfits as defined. Therefore, a backfit analysis is not required.

Small Business Regulatory Enforcement Fairness Act

In accordance with the Small Business Regulatory Enforcement Fairness Act of 1996, the NRC has determined that this action is not a major rule and has verified this determination with the Office of Information and Regulatory Affairs, Office of Management and Budget.

List of Subjects in 10 CFR Part 72

Administrative practice and procedure, Criminal penalties, Manpower training programs, Nuclear materials, Occupational safety and health, Penalties, Radiation protection, Reporting and recordkeeping

requirements, Security measures, Spent fuel, Whistleblowing.

For the reasons set out in the preamble and under the authority of the Atomic Energy Act of 1954, as amended; the Energy Reorganization Act of 1974, as amended; and 5 U.S.C. 552 and 553; the NRC is adopting the following amendments to 10 CFR Part 72.

PART 72—LICENSING REQUIREMENTS FOR THE INDEPENDENT STORAGE OF SPENT NUCLEAR FUEL AND HIGH-LEVEL RADIOACTIVE WASTE

1. The authority citation for Part 72 continues to read as follows:

Authority: Secs. 51, 53, 57, 62, 63, 65, 69, 81, 161, 182, 183, 184, 186, 187, 189, 68 Stat. 929, 930, 932, 933, 934, 935, 948, 953, 954, 955, as amended, sec. 234, 83 Stat. 444, as amended (42 U.S.C. 2071, 2073, 2077, 2092, 2093, 2095, 2099, 2111, 2201, 2232, 2233, 2234, 2236, 2237, 2238, 2282); sec. 274, Pub. L. 86–373, 73 Stat. 688, as amended (42 U.S.C. 2021); sec. 201, as amended, 202, 206, 88 Stat. 1242, as amended, 1244, 1246 (42 U.S.C. 5841, 5842, 5846); Pub. L. 95–601, sec. 10, 92 Stat. 2951, as amended by Pub. L. 102–486, sec. 7902, 106 Stat. 3123 (42 U.S.C. 5851); sec. 102, Pub. L. 91–190, 83 Stat. 853 (42 U.S.C. 4332); secs. 131, 132, 133, 135, 137, 141, Pub. L. 97–425, 96 Stat. 2229, 2230, 2232, 2241, sec. 148, Pub. L. 100–203, 101 Stat. 1330–235 (42 U.S.C. 10151, 10152, 10153, 10155, 10157, 10161, 10168).

Section 72.44(g) also issued under secs. 142(b) and 148(c), (d), Pub. L. 100–203, 101 Stat. 1330–232, 1330–236 (42 U.S.C. 10162(b), 10168(c), (d)). Section 72.46 also issued under sec. 189, 68 Stat. 955 (42 U.S.C. 2239); sec. 134, Pub. L. 97–425, 96 Stat. 2230 (42 U.S.C. 10154). Section 72.96(d) also issued under sec. 145(g), Pub. L. 100–203, 101 Stat. 1330–235 (42 U.S.C. 10165(g)). Subpart J also issued under secs. 2(2), 2(15), 2(19), 117(a), 141(h), Pub. L. 97–425, 96 Stat. 2202, 2203, 2204, 2222, 2244, (42 U.S.C. 10101, 10137(a), 10161(h)). Subparts K and L are also issued under sec. 133, 98 Stat. 2230 (42 U.S.C. 10153) and sec. 218(a), 96 Stat. 2252 (42 U.S.C. 10198).

2. In § 72.214, Certificate of Compliance 1014 is revised to read as follows:

§ 72.214 List of approved spent fuel storage casks.

* * * * *

Certificate Number: 1014
Initial Certificate Effective Date: June 1, 2000
Amendment Number 1 Effective Date: June 10, 2002
SAR Submitted by: Holtec International
SAR Title: Final Safety Analysis Report for the HI-STORM 100 Cask System
Docket Number: 72–1014
Certificate Expiration Date: June 1, 2020
Model Number: HI-STORM 100

* * * * *

Dated at Rockville, Maryland, this 13th day of March, 2002.

For the Nuclear Regulatory Commission.

William D. Travers,

Executive Director for Operations.

[FR Doc. 02–7320 Filed 3–26–02; 8:45 am]

BILLING CODE 7590–01–P

DEPARTMENT OF TRANSPORTATION

Federal Aviation Administration

14 CFR Part 39

[Docket No. 2002–NE–05–AD; Amendment 39–12684; AD 2002–06–07]

RIN 2120–AA64

Airworthiness Directives; General Electric Company CF6–80E1 Series Turbofan Engines

AGENCY: Federal Aviation Administration, DOT.

ACTION: Final rule; request for comments.

SUMMARY: This amendment adopts a new airworthiness directive (AD) that is applicable to General Electric Company (GE) CF6–80E1 series turbofan engines, installed on Airbus Industrie A330 series airplanes. This action requires initial and repetitive pressure testing of the directional pilot valve (DPV) assembly, with replacement of DPV assemblies that fail the pressure test, or, replacing the DPV assembly without performing pressure testing, with a serviceable DPV assembly, or, deactivating the fan reverser for no longer than 10 days until replacement of the DPV assembly is done. This amendment is prompted by a review of thrust reverser safety analyses following a report of inadvertent thrust reverser deployment on another make and model engine. The actions specified in this AD are intended to prevent inadvertent thrust reverser deployment, which, if it occurred in-flight, could result in loss of control of the airplane.

DATES: Effective May 1, 2002. The incorporation by reference of certain publications listed in the rule is approved by the Director of the Federal Register as of May 1, 2002.

Comments for inclusion in the Rules Docket must be received on or before May 28, 2002.

ADDRESSES: Submit comments in triplicate to the Federal Aviation Administration (FAA), New England Region, Office of the Regional Counsel, Attention: Rules Docket No. 2002–NE–05–AD, 12 New England Executive Park, Burlington, MA 01803–5299. Comments may be inspected at this location, by appointment, between 8 a.m. and 4:30 p.m., Monday through Friday, except Federal holidays. Comments may also be sent via the Internet using the following address: ''*9-ane-adcomment@faa.gov*''. Comments sent via the Internet must contain the docket number in the subject line.

The service information referenced in this AD may be obtained from Middle River Aircraft Systems, Mail Point 46, 103 Chesapeake Park Plaza, Baltimore, MD, 21220–4295, attn: Warranty Support, telephone: (410) 682–0094, fax: (410) 682–0100. This information may be examined, by appointment, at the FAA, New England Region, Office of the Regional Counsel, 12 New England Executive Park, Burlington, MA; or at the Office of the Federal Register, 800 North Capitol Street, NW., suite 700, Washington, DC.

FOR FURTHER INFORMATION CONTACT: Karen Curtis, Aerospace Engineer, Engine Certification Office, FAA, Engine and Propeller Directorate, 12 New England Executive Park, Burlington, MA 01803–5299; telephone (781) 238–7192; fax (781) 238–7199.

SUPPLEMENTARY INFORMATION: The FAA received a report of inadvertent thrust reverser deployment on a Pratt & Whitney powered Airbus Industrie A300–600 series airplane. Following that event, the FAA reviewed thrust reverser safety analyses on other make and model engines, including GE CF6–80E1 series turbofan engines that are used on Airbus Industrie A330 series airplanes. A review of thrust reverser actuation system (TRAS) shop findings and component failure rate data, test data, and system safety analyses revealed that a hidden failure mode involving the directional pilot valve (DPV) assembly exists. The DPV assembly controls the direction of the operation of the center drive unit when the TRAS is activated. If high pressure downstream of the pressure regulating and shutoff valve (PRSOV) exists in combination with a leaking DPV assembly, an inadvertent deployment could occur. High pressure downstream of the PRSOV can be caused by auto-restow of the thrust reverser, PRSOV open failures, or significant PRSOV leakage. PRSOV open failures and significant PRSOV leakage are detected by the DPV assembly pressure switch. DPV assembly open failures and significant DPV assembly leakage are detected by the inability to stow the reverser. However, there exists a range of DPV assembly leakage rates from a closed DPV assembly which are not detectable during normal operation.