#### FOR FURTHER INFORMATION CONTACT:

James Wade, OFPP, jwade@ omb.eop.gov.

## Background

Value Engineering (VE) is a management technique that is used to analyze activities and identify alternative processes for completing the activities at a lower cost. Industry first developed VE during World War II as a means of continuing production despite shortages of critical materials. The Federal Government subsequently adopted VE as a mechanism to improve efficiency. Policies adding the use of VE are set forth in OMB Circular A–131 at https://www.whitehouse.gov/sites/whitehouse.gov/files/omb/circulars/A131/a131-122013.pdf.

Use of VE supports the Administration's efforts to emphasize critical thinking and analysis instead of compliance activity and documentation. Although several Federal agencies have reported life-cycle savings in a broad range of acquisition programs, including defense, transportation, and construction projects, overall usage of VE by federal agencies has been limited. OFPP believes agency workforce awareness and consideration of VE can be improved by redirecting agency resources away from compliance reporting and towards information sharing with other agencies on use of the tool through the Acquisition Gateway (https://hallways.cap.gsa.gov/ login-information). The Gateway provides federal buyers with a forum for improving government acquisition. The Acquisition Innovation Hub within the Gateway facilitates information sharing with tools and resources for acquisition professionals and other stakeholders. These actions are called for by OMB Memorandum M-17-26, "Reducing Burden for Federal Agencies by Rescinding and Modifying OMB Memoranda." Memorandum M-17-26 was designed to eliminate inefficiencies created by past OMB direction and improve the efficiency of government operations.

Accordingly, OFPP proposes the following changes to Circular A–131, as revised in December 2013:

1. Replace section 8, entitled "Reports to OMB" with the following new section 8: Information Sharing. Agencies are encouraged to share best practices, case studies and other information about their experience using VE on the Acquisition Gateway (https://hallways.cap.gsa.gov/login-information). The Gateway connects federal buyers with resources and tools to improve acquisition throughout the government. The Acquisition Innovation Hub within the Gateway facilitates information sharing between acquisition professionals and other

stakeholders. Sharing information on the Hub can help build greater awareness of VE and accelerate the pace of innovation and other benefits that can come from the use of this management tool.

- 2. Make the following conforming changes: a. Delete paragraph f. from section 7, which refers to reporting.
- b. Delete the Attachment to the Circular, which provides a format for reporting to OMB.

For a copy of OMB Circular A–131, go to https://www.whitehouse.gov/sites/whitehouse.gov/ files/omb/circulars/A131/a131-122013.pdf.

Although public comment is not required in the development of these changes, OMB welcomes input on the proposed amendments to the Circular described above and will consider feedback prior to finalizing changes to the Circular.

#### Lesley A. Field,

Deputy Administrator for Federal Procurement Policy.

[FR Doc. 2018–04445 Filed 3–5–18; 8:45 am]

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# NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-334 and 50-412; NRC-2018-0041]

FirstEnergy Nuclear Operating Company; Beaver Valley Power Station; Unit Nos. 1 and 2; Use of Optimized ZIRLO™ Fuel Rod Cladding

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Exemption; issuance.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC) is issuing an exemption in response to an April 9, 2017, request from FirstEnergy Nuclear Operating Company (FENOC), in order to use Optimized ZIRLO<sup>TM</sup> fuel rod cladding at the Beaver Valley Power Station, Unit Nos. 1 and 2 (Beaver Valley).

**DATES:** The exemption was issued on March 6, 2018.

ADDRESSES: Please refer to Docket ID NRC–2018–0041 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

• Federal Rulemaking website: Go to http://www.regulations.gov and search for Docket ID NRC-2018-0041. Address questions about NRC dockets to Jennifer Borges; telephone: 301-287-9127; email: Jennifer.Borges@nrc.gov. For technical questions, contact the

individual listed in the FOR FURTHER INFORMATION CONTACT section of this document.

- NRC's Agencywide Documents Access and Management System (ADAMS): You may obtain publiclyavailable documents online in the ADAMS Public Documents collection at http://www.nrc.gov/reading-rm/ adams.html. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document. In addition, for the convenience of the reader, the ADAMS accession numbers are provided in a table in the "Availability of Documents" section of this document.
- NRC's PDR: You may examine and purchase copies of public documents at the NRC's PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

## FOR FURTHER INFORMATION CONTACT:

Tanya E. Hood, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001; telephone: 301–415– 1387, email: *Tanya.Hood@nrc.gov*.

## SUPPLEMENTARY INFORMATION:

## I. Background

The FirstEnergy Nuclear Operating Company (FENOC) is the holder of Renewed Facility Operating License Nos. 50-334 and 50-412, which authorize operation of Beaver Valley. The licenses provide, among other things, that the facilities are subject to all rules, regulations, and orders of the NRC now or hereafter in effect. The facilities consist of pressurized-water reactors located in Shippingport Borough on the Ohio River in Beaver County, Pennsylvania. The ZIRLO® corrosion model was based on a model originally developed for zircalov-4 cladding. As utilities moved to increased fuel thermal duty associated with higher peaking factors, uprated core power, and longer cycle lengths, cladding corrosion has become one of the important factors in assessing the potential for increased fuel thermal duty.

#### II. Request/Action

Pursuant to title 10 of the *Code of Federal Regulations* (10 CFR) section 50.12, "Specific exemptions," the

licensee requested, by letter dated April 9, 2017 (ADAMS Accession No. ML17100A269), an exemption from § 50.46, "Acceptance criteria for emergency core cooling systems [ECCS] for light-water nuclear power reactors," and 10 CFR part 50, appendix K, "ECCS Evaluation Models," to allow the use of Optimized ZIRLO<sup>TM</sup> fuel rod cladding for future core reload applications. The regulations in § 50.46 contain acceptance criteria for the ECCS for reactors fueled with zircaloy or ZIRLO® fuel rod cladding material. In addition, 10 CFR part 50, appendix K, requires that the Baker-Just equation be used to predict the rates of energy release, hydrogen concentration, and cladding oxidation from the metal/water reaction. The Baker-Just equation assumes the use of a zirconium alloy different from Optimized ZIRLO<sup>TM</sup> material. Therefore, an exemption to § 50.46 and 10 CFR part 50, appendix K, is required to support the use of Optimized ZIRLO<sup>TM</sup> fuel rod cladding at Beaver Vallev.

The exemption request relates solely to the specific types of cladding material specified in these regulations for use in light-water reactors (*i.e.*, fuel rods with zircaloy or ZIRLO® cladding). This request will provide for the application of the acceptance criteria of § 50.46 and 10 CFR, part 50, appendix K, to fuel assembly designs using Optimized ZIRLOTM fuel rod cladding.

### III. Discussion

Pursuant to § 50.12, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR part 50 when: (1) The exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security; and (2) when special circumstances are present. Under § 50.12(a)(2), special circumstances include, among other things, when application of the specific regulation in the particular circumstance would not serve, or is not necessary to achieve, the underlying purpose of the rule.

The Optimized ZIRLO<sup>TM</sup> fuel cladding is different from standard ZIRLO® in two respects: (1) The tin content is lower and (2) the microstructure is different. This difference in tin content and microstructure can lead to differences in some material properties. Westinghouse Electric Company (Westinghouse), the manufacturer of Optimized ZIRLO<sup>TM</sup> fuel rod cladding, has committed to provide irradiated data and validate fuel performance models ahead of burnups

achieved in batch application (*i.e.*, a group of fuel assemblies).

The NRC staff's safety evaluation of Optimized ZIRLO<sup>TM</sup> (WCAP–12610–P–A & CENPD–404–P–A) dated June 10, 2005 (ADAMS Package Accession No. ML051670395), included ten conditions and limitations. The NRC staff reviewed FENOC's April 9, 2017, application against these specific conditions and concluded that the licensee is in compliance with all of the applicable conditions, with the exception of Conditions 6 and 7.

Conditions 6 and 7 relate to validating in-reactor performance and fuel performance models based on lead test assembly data obtained ahead of batch application. Westinghouse provided additional information from irradiation programs to comply with Conditions 6 and 7 of the NRC staff's safety evaluation, by letters dated February 25, 2013 (ADAMS Accession No. ML13070A188), and February 9, 2015 (ADAMS Accession No. ML15051A427), demonstrating compliance with these two conditions.

One of the main objectives of the ongoing Westinghouse creep (growth) program was to confirm the adequacy of the Westinghouse Performance Analysis and Design Model creep models for Optimized ZIRLO<sup>TM</sup> and verify that the steady state irradiation creep rate is the same in tension and compression. Based upon the supporting data provided by Westinghouse in Figures 3 through 6 of WCAP-12610-P-A & CENPD-404-P-A Addendum 1–A (ADAMS Accession No. ML13070A189), the NRC staff determined that the creep models are adequate for the first operating cycle where the fuel rod cladding is predominately in compressive creep.

The NRC staff performed its review of Conditions 6 and 7 in a letter dated August 3, 2016 (ADAMS Accession No. ML16173A354). The NRC staff determined that the data provided in Westinghouse letters dated February 25, 2013, and February 9, 2015, satisfy Conditions 6 and 7. Therefore, licensees no longer need to provide additional data when referencing WCAP-12610-P-A & CENPD-404-P-A, Addendum 1-A, "Optimized ZIRLOTM," July 2006, in future license amendment requests.

The licensee provided documentation of its compliance with the Westinghouse topical report WCAP–12610–P–A & CENPD–404–P–A Addendum 1–A commitments in its application. Based on that documentation and the information contained in Westinghouse's revised compliance letters dated February 25, 2013, and February 9, 2015, the NRC staff finds the licensee's compliance

with safety evaluation Conditions 6 and 7 is acceptable.

A. The Exemption Is Authorized by Law

This exemption would allow the use of Optimized ZIRLOTM fuel rod cladding material at Beaver Valley. As stated above, § 50.12 allows the NRC to grant exemptions from the requirements of 10 CFR part 50. The fuel that will be irradiated at Beaver Valley contains cladding material that does not conform to the cladding material that is explicitly defined in 10 CFR 50.46 and implicitly defined in 10 CFR part 50, appendix K. However, the criteria of these regulations will continue to be satisfied for the operation of the Beaver Valley cores containing Optimized ZIRLO<sup>TM</sup> fuel cladding. The NRC staff has determined that granting the licensee's proposed exemption would not result in a violation of the Atomic Energy Act of 1954, as amended, or the Commission's regulations. Therefore, the exemption is authorized by law.

B. The Exemption Presents No Undue Risk to Public Health and Safety

The objectives of  $\S 50.46(b)(2)$  and (b)(3) and 10 CFR part 50, appendix K, section I.A.5, are to ensure that cladding oxidation and hydrogen generation are appropriately limited during loss-ofcoolant accidents and conservatively accounted for in ECCS evaluation models. As previously documented in the NRC staff's safety evaluation of topical reports submitted by Westinghouse, dated June 10, 2005, and subject to compliance with the specific conditions of approval established in the safety evaluation, the NRC staff found that Westinghouse demonstrated the applicability of the ECCS acceptance criteria to Optimized ZIRLOTM. The NRC staff concluded that oxidation measurements provided by the licensee in the letter from Westinghouse to the NRC, dated November 2007 (ADAMS Accession No. ML073130560), illustrate that oxide thickness and associated hydrogen pickup for Optimized ZIRLO<sup>TM</sup> at any given burnup would be less than those of both zircaloy-4 and

The NRC staff previously found that metal-water reaction tests performed by Westinghouse on Optimized ZIRLOTM (see appendix B of WCAP-12610-P-A & CENPD-404-P-A Addendum 1-A) demonstrate conservative reaction rates relative to the Baker-Just equation. Thus, the NRC staff determined that the application of appendix K, section I.A.5, is not necessary to achieve the underlying purpose of the rule in these circumstances. Since these evaluations demonstrate that the underlying

purpose of the rule will be met, there will be no undue risk to the public health and safety. The facility operating licenses require that reload cores be operated in accordance with the operating limits specified in the technical specifications and core operating limits report. Thus, the granting of this exemption request will not pose an undue risk to public health and safety.

# C. The Exemption Is Consistent With the Common Defense and Security

The exemption request would allow the licensee to use an improved fuel rod cladding material. In its letter dated April 9, 2017, the licensee stated that all the requirements and acceptance criteria will be maintained. The licensee is required to handle and control special nuclear material in these assemblies in accordance with its approved procedures. Use of Optimized ZIRLOTM fuel rod cladding in the Beaver Valley cores will not adversely affect plant operations. Therefore, the NRC staff has determined that this exemption does not adversely impact common defense and security.

### D. Special Circumstances

Special circumstances, in accordance with § 50.12(a)(2)(ii), are present whenever application of the regulation

in the particular circumstances is not necessary to achieve the underlying purpose of the rule. The underlying purpose of § 50.46 and 10 CFR part 50, appendix K, is to establish acceptance criteria for ECCS performance. The regulations ensure that nuclear power reactors fueled with uranium oxide pellets within zircaloy or ZIRLO® cladding must be provided with an ECCS designed to provide core cooling following postulated loss-of-coolant accidents. Westinghouse demonstrated in its NRC-approved topical report WCAP-12610-P-A & CENPD-404-P-A Addendum 1-A that ECCS effectiveness will not be adversely affected by a change from zircaloy or ZIRLO® clad fuel to Optimized ZIRLOTM clad fuel. Normal safety analyses performed prior to core reload will confirm that there is no adverse impact on ECCS performance. Therefore, since the underlying purposes of § 50.46 and 10 CFR part 50, appendix K, are achieved through the use of Optimized ZIRLOTM fuel rod cladding material, the special circumstances required by § 50.12(a)(2)(ii) for the granting of an exemption exist.

#### E. Environmental Considerations

The NRC staff determined that the exemption discussed herein meets the eligibility criteria for the categorical

exclusion set forth in § 51.22(c)(9) because it is related to a requirement concerning the installation or use of a facility component located within the restricted area, as defined in 10 CFR part 20, and the granting of this exemption involves: (1) No significant hazards consideration, (2) no significant change in the types or a significant increase in the amounts of any effluents that may be released offsite, and (3) no significant increase in individual or cumulative occupational radiation exposure. Therefore, in accordance with § 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the NRC staff's consideration of this exemption request. The basis for the NRC staff's determination is discussed in an evaluation of the requirements of § 51.22(c)(9) in the proposed no significant hazards consideration determination for the associated amendment as published in the Federal Register on July 18, 2017 (82 FR 32881).

#### IV. Availability of Documents

The NRC is making the documents identified below available to interested persons through one or more of the following methods, as indicated. To access documents related to this action, see ADDRESSES Section of this document.

Document	ADAMS accession No.
Beaver Valley Power Station, Unit Nos. 1 and 2, "License Amendment Request to Modify Technical Specifications 4.2.1 and 5.6.3 and a 10 CFR 50.12 Exemption Request to Implement Optimized ZIRLO <sup>TM</sup> Fuel Rod Cladding" (April 9, 2017).	ML17100A269
Westinghouse—Final Safety Evaluation for Addendum 1 to Topical Report WCAP-12610-P-A & CENPD-404-P-A Addendum 1-A, "Optimized ZIRLOTM" (June 10, 2005).	ML051670395
Westinghouse—LTR-NRC-13-6, NP-Attachment—SER Compliance with WCAP-12610-P-A & CENPD-404-P-A Addendum 1-A "Optimized ZIRLO <sup>TM</sup> " (February 25, 2013).	ML13070A188
Westinghouse—LTR-NRC-15-7, Submittal of Responses to Draft RAIs and Revisions to Select Figures in LTR-NRC-13-6 to Fulfill Conditions 6 and 7 of the Safety Evaluation for WCAP-12610-P-A & CENPD-404-P-A, Addendum 1-A (February 9, 2015).	ML15051A427
Westinghouse—LTR-NRC-13-6, NP-Attachment, SER Compliance of WCAP-12610-P-A & CENPD-404-P-A Addendum 1-A (Non-Proprietary) "Optimized ZIRLO <sup>TM</sup> " (February 2013).	ML13070A189
Satisfaction of Conditions 6 & 7 of the Safety Evaluation for WCAP-12610-P-A and CENPD-404-P-A, Addendum 1-A "Optimized ZIRLO <sup>TM</sup> " Topical Report.	ML16173A354
SER Compliance with WCAP-12610-P-A & CENPD-404-P-A Addendum 1-A, "Optimized ZIRLO <sup>TM</sup> " (Non-Proprietary) (November 2007).	ML073130560

#### V. Conclusion

Accordingly, the Commission has determined that pursuant to § 50.12, the exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. Also, special circumstances are present.

Therefore, the Commission hereby grants FENOC an exemption from the requirements of § 50.46 and 10 CFR part 50, appendix K, to allow the use of Optimized ZIRLO<sup>TM</sup> fuel rod cladding material at Beaver Valley.

Dated at Rockville, Maryland, this 1st day of March 2018.

For the Nuclear Regulatory Commission. **Joseph G. Giitter**,

Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

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