By letter dated February 22, 1994, the NRC approved the use of the master curve approach for the Zion Nuclear Power Station, Units 1 and 2, and the RT_{NDT} value is -26 °F for WF-70 weld metal. The exemption approval for the Zion station also stated that other procedures for determination of RT_{NDT} may serve as acceptable alternatives to NB-2331 contingent on staff review and approval. The staff acceptance of the alternative procedure in that evaluation was based, in part, on the analysis of a significant amount of fracture toughness data for the WF-70 weld metal. Therefore, since TMI-1 used the same weld metal as Zion and the data considered for the Zion exemption resulted in a more representative RT_{NDT} value, the TMI-1 use of the master curve approach for WF-70 weld metal is acceptable.

In summary, the underlying purpose of 10 CFR 50.61 is to ensure that the RPV is adequately protected from PTS. Application of the master curve approach to determine the unirradiated $RT_{\rm NDT}$ value for weld metal WF–70 is acceptable because the master curve approach is more appropriate for material with low upper-shelf behavior like WF–70 weld metal.

Therefore, pursuant to 10 CFR 50.12(a)(2)(ii), application of the master curve approach to determine the unirradiated RT_{NDT} value for weld metal WF–70 would continue to achieve the underlying purpose of the rule, and application of the definition of $RT_{NDT(U)}$ in 10 CFR 50.61(a)(5) in these circumstances is not necessary to achieve that purpose.

4.0 Conclusion

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a), the exemptions are authorized by law, will not endanger life or property or common defense and security, and are, otherwise, in the public interest. Also, special circumstances are present. Therefore, the Commission hereby grants AmerGen Energy Company, LLC exemptions from the requirements of 10 CFR part 50, Appendix G, and 10 CFR part 50, § 50.61(a)(5), for TMI-1.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will not have a significant effect on the quality of the human environment (66 FR 45874).

This exemption is effective upon

Dated at Rockville, Maryland, this 30th day of August 2001.

For the Nuclear Regulatory Commission. **John A. Zwolinski**,

Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 01–22514 Filed 9–6–01; 8:45 am] BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 72-8]

Calvert Cliffs Nuclear Power Plant; Notice of Docketing of the Materials License SNM-2505; Amendment Application for the Calvert Cliffs Independent Spent Fuel Storage Installation

By letter dated July 26, 2001, Calvert Cliffs Nuclear Power Plant, Inc. (CCNPP), submitted an application to the Nuclear Regulatory Commission (NRC or the Commission) in accordance with 10 CFR part 72 requesting an amendment of the Calvert Cliffs independent spent fuel storage installation (ISFSI) license (SNM-2505) for the ISFSI located in Calvert County, Maryland. CCNPP is requesting Commission approval to amend SNM-2505 to reflect revised fuel assembly integrity analysis as described in the Safety Analysis Report. CCNPP proposed changes to Technical Specification 2.3 to remove the 15-inch drop height limit and require inspection after any drop of a dry shielded canister. CCNPP also proposed a change to Technical Specification 6.3 to revise the reference to a semi-annual environmental reporting period to be consistent with the annual reporting requirements of 10 CFR 50.36a(2).

This application was docketed under 10 CFR part 72; the ISFSI Docket No. is 72–8 and will remain the same for this action. The amendment of an ISFSI license is subject to the Commission's approval.

The Commission may issue either a notice of hearing or a notice of proposed action and opportunity for hearing in accordance with 10 CFR 72.46(b)(1) or, if a determination is made that the amendment does not present a genuine issue as to whether public health and safety will be significantly affected, take immediate action on the amendment in accordance with 10 CFR 72.46(b)(2) and provide notice of the action taken and an opportunity for interested persons to request a hearing on whether the action should be rescinded or modified.

The NRC maintains an Agencywide Documents Access and Management System (ADAMS), which provides text and image files of NRC's public documents. These documents may be accessed through NRC's Public Electronic Reading Room on the Internet at http://www.nrc.gov/nrc/adams/index.html. If you do not have access to ADAMS or if there are problems in accessing documents located in ADAMS, contact the NRC Public Document Room Reference staff at 1—800—397—4209, 301—415—4737, or by email to pdr@nrc.gov.

Dated at Rockville, Maryland, this 29th day of August 2001.

For the Nuclear Regulatory Commission. **E. William Brach**,

Director, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards. [FR Doc. 01–22515 Filed 9–6–01; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-334 and 50-412]

FirstEnergy Nuclear Operating Company, et al., Beaver Valley Power Station, Unit Nos. 1 and 2; Notice of Issuance of Amendment to Facility Operating License

The U.S. Nuclear Regulatory
Commission (Commission) has issued
Amendment Nos. 241 and 121 to
Facility Operating License Nos. DPR-66
and NPF-73, respectively, issued to
FirstEnergy Nuclear Operating
Company, et al. (the licensee), which
revised the Technical Specifications
(TSs) and authorized revisions to the
Updated Final Safety Analysis Report
(UFSAR) for operation of Beaver Valley
Power Station, Unit Nos. 1 and 2,
located in Shippingport, Pennsylvania.
The amendment is effective as of the
date of issuance.

The amendment authorized revisions to the BVPS-1 and 2 UFSAR designbasis fuel handling accident (FHA) dose consequence analyses. The amendment also revised the BVPS-1 and 2 TSs associated with the requirements for handling irradiated fuel assemblies in the reactor containment and fuel building and the TS requirements associated with ensuring that UFSAR safety analysis assumptions are met for a postulated FHA. The term "recently irradiated" fuel is defined in the applicable TS Bases as "fuel that has occupied part of a critical reactor core within the previous 100 hours." The purpose of the addition of the term "recently irradiated" throughout the TSs is to establish a point where operability of those systems typically used to mitigate the consequences of an FHA is no longer required to meet the