

must be accompanied or preceded by an LTP to be submitted for NRC approval. If found acceptable by the NRC staff, the LTP is approved by license amendment, subject to such conditions and limitations as the NRC staff deems appropriate and necessary. CYAPC submitted the proposed LTP for HNP by application dated July 7, 2000. In accordance with 10 CFR 20.1405 and 10 CFR 50.82(a)(9)(iii), the NRC provided notice to individuals in the vicinity of the site that the NRC was in receipt of the HNP LTP and would accept comments from affected parties (65 FR 51345). In accordance with 10 CFR 50.82(a)(9)(iii), the NRC is hereby providing notice that NRC staff will conduct a meeting to discuss the HNP LTP.

The HNP LTP (ADAMS Accession Number ML003735143) may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, and is accessible electronically from the ADAMS Public Library component on the NRC Web site, <http://www.nrc.gov> (the Public Electronic Reading Room). The LTP may also be viewed at the CYAPC Web site at www.connyankee.com.

For further information, contact: Mr. Louis L. Wheeler by mail, Mail Stop O-7-C2, Project Directorate IV & Decommissioning, Division of Licensing Project Management, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555-0001; telephone 301-415-1444; or e-mail dxw@nrc.gov.

Dated at Rockville, Maryland, this 28th day of September 2000.

For the Nuclear Regulatory Commission.

Michael T. Masnik,

Chief, Decommissioning Section, Project Directorate IV and Decommissioning, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-368]

Entergy Operations, Inc.; Arkansas Nuclear One, Unit 2 Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from 10 CFR Part 50, Appendix J, for Facility Operating License No. NPF-6, issued to

Entergy Operations, Inc. (the licensee), for operation of Arkansas Nuclear One, Unit 2 (ANO-2), located in Pope County, Arkansas.

Environmental Assessment

Identification of the Proposed Action

The proposed action would provide a one-time exemption to Entergy Operations, Inc. from the requirements of Title 10 of the Code of Federal Regulations (10 CFR) Part 50, Appendix J, "Primary Reactor Containment Leakage Testing For Water-Cooled Power Reactors," which requires that licensees of all power reactors conduct integrated leakage rate tests (ILRT) under conditions representing design basis loss-of-coolant accident containment peak pressure. The licensee requires an exemption in order to conduct the ILRT at the same pressure that is used for the structural integrity test (SIT).

The proposed action is in accordance with the licensee's application for exemption dated June 29, 2000.

The Need for the Proposed Action

The ANO-2 steam generators (SGs) are scheduled for replacement during the fall of 2000. The replacement SGs (RSGs) will require that an access opening be cut in the containment building structure. Upon closure of the structure, an ILRT will be required to test for primary containment leakage integrity.

The ANO-2 containment building was originally designed and tested for an internal pressure of 54 psig. The ANO-2 containment building has recently been reevaluated, to address the containment post-accident response resulting from the RSGs, for an increase in accident pressure to 58 psig with a design pressure of 59 psig, and shown to be acceptable as discussed in a letter to the NRC dated November 3, 1999, as revised by a letter dated June 29, 2000. As a result of this increase, an SIT will be performed to evaluate the ANO-2 containment building for the change in containment design pressure. The purpose of the SIT is to verify that the containment building structure can safely carry design loads and that the structural behavior is similar to that predicted by analysis. The post-RSG SIT will be performed at 68 psig (1.15 times the revised design pressure). The licensee would like to also perform the ILRT concurrently with the post-RSG SIT, at the SIT pressure of 68 psig, in order to recover approximately 30 hours of projected plant outage time. However, Appendix J requires that the ILRT be conducted at a pressure representing the

design basis loss-of-coolant accident containment peak pressure, which is 58 psig. Hence, the need for the proposed exemption.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that the proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not involve any historic sites. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (*i.e.*, the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the "Final Environmental Statement Related to the Operation of Arkansas Nuclear One, Unit 2," dated June 1977.

Agencies and Persons Consulted

In accordance with its stated policy, on September 7, 2000, the staff consulted with the Arkansas State official, Bernie Beville of the Arkansas Department of Health, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the

NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated June 29, 2000, which may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the ADAMS Public Library component on the NRC Web site, <http://www.nrc.gov> (the Electronic Reading Room).

Dated at Rockville, Maryland, this 28th day of September 2000.

For the Nuclear Regulatory Commission.

Mohan C. Thadani,

Acting Chief, Section 1, Project Directorate IV-1 & Decommissioning Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Meeting of the Ad Hoc Subcommittee; Revised

The ACRS Ad Hoc Subcommittee meeting scheduled for October 10-13, 2000 has been extended to Saturday, October 14, 2000, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland, 8:30 a.m. until 12 Noon to discuss proposed comments and recommendations on the technical merits of the Differing Professional Opinion Issues associated with steam generator tube integrity. Notice of this meeting was previously published in the **Federal Register** on Wednesday, September 20, 2000 (65 FR 56945). All other items pertaining to this meeting remains the same as previously published.

For further information contact either Mr. Sam Duraiswamy (telephone 301-415-7364) or Ms. Undine Shoop (telephone 301-415-8086) between 7:30 a.m. and 4:15 p.m. (EDT).

Dated: September 28, 2000.

James E. Lyons,

Associate Director for Technical Support, ACRS/ACNW.

[FR Doc. 00-25459 Filed 10-3-00; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Meeting of the Subcommittee on Plant License Renewal

The ACRS Subcommittee on Plant License Renewal will hold a meeting on October 19-20, 2000, Room T-2B1, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Thursday, October 19, 2000—8 a.m. until the conclusion of business.

The Subcommittee will review drafts of the Standard Review Plan for license renewal and the Generic Aging Lessons Learned (GALL) Report sections 2, 3, and 4.

Friday, October 20, 2000—8 a.m. until the conclusion of business.

The Subcommittee will review drafts of GALL Report sections 5 through 8, the associated Regulatory Guide, and Nuclear Energy Institute (NEI) 95-10, "Industry Guideline For Implementing The Requirements of 10 CFR Part 54—The License Renewal Rule."

The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of the NRC staff, the Nuclear Energy Institute, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting

has been canceled or rescheduled, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor, can be obtained by contacting the cognizant ACRS staff engineer, Mr. Noel F. Dudley (telephone 301/415-6888) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: September 28, 2000

James E. Lyons,

Associate Director for Technical Support, ACRS/ACNW.

[FR Doc. 00-25460 Filed 10-3-00; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Reactor Fuels; Notice of Meeting

The ACRS Subcommittee on Reactor Fuels will hold a meeting on October 18, 2000, Room T-2B1, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, October 18, 2000—8:30 a.m. until the conclusion of business.

The Subcommittee will discuss the status of the staff's effort regarding the draft report of a technical study of Spent Fuel Pool Accident Risk at Decommissioning Nuclear Power Plants, and related matters. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be