

himself or herself by stating their name, city and state of residence, and stating whether they have any affiliation (such as employment, consultancy, or membership) with any of the parties (USEC or the NRC).

C. Submitting a Request To Make an Oral Limited Appearance Statement

Persons wishing to make an oral statement who have submitted a timely written request to do so will be given priority over those who have not filed such a request. To be considered timely, a written request to make an oral statement must either be mailed, faxed, or sent by e-mail so as to be received by 5 p.m. EST on January 2, 2007. Written requests to make an oral statement should be submitted to:

Mail: Office of the Secretary, Rulemakings and Adjudications Staff, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

Fax: (301) 415-1101 (verification (301) 415-1966).

E-mail: hearingdocket@nrc.gov.

In addition, using the same method of service, a copy of the written request to make an oral statement should be sent to the Chairman of this Licensing Board as follows:

Mail: Administrative Judge Lawrence G. McDade, c/o: Debra Wolf, Esq. Law Clerk, Atomic Safety and Licensing Board Panel, Mail Stop T-3 F23, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555-0001.

Fax: (301) 415-5599 (verification (301) 415-6094).

E-mail: daw1@nrc.gov.

D. Submitted Written Limited Appearance Statements

A written limited appearance statement may be submitted to the Board regarding this proceeding at any time, either in lieu of or in addition to any oral statement. Such statements should be sent to the Office of the Secretary using the methods prescribed above, with a copy to the Licensing Board Chairman.

E. Availability of Documentary Information Regarding the Proceeding

Documents relating to this proceeding are available for public inspection at the Commission's Public Document Room (PDR), located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, or electronically from the publicly available records component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (Electronic Reading Room). Persons who do not have access to ADAMS or who

encounter problems in accessing the documents located in ADAMS should contact the NRC PDR reference staff by telephone at (800) 397-4209 or (301) 415-4737, or by e-mail to pdr@nrc.gov.

F. Scheduling Information Updates

Updated/revised scheduling information regarding the limited appearance session can be found on the NRC Web site at <http://www.nrc.gov/public-involve/public-meetings/index.cfm> or by calling (800) 368-5642, extension 5036, or (301) 415-5036.

Dated in Rockville, Maryland, on November 17, 2006.

For the Atomic Safety and Licensing Board.²

Lawrence G. McDade,
Chairman, Administrative Judge.

[FR Doc. E6-19839 Filed 11-22-06; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Notice of Availability of the Final License Renewal Interim Staff Guidance—LR-ISG-2006-01: Plant-Specific Aging Management Program for Inaccessible Areas of Boiling Water Reactor (BWR) Mark I Steel Containment Drywell Shell

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of availability.

SUMMARY: The NRC is issuing its Final License Renewal Interim Staff Guidance LR-ISG-2006-01. This LR-ISG provides interim guidance to applicants for license renewal for a plant with a BWR Mark I steel containment to provide a plant-specific aging management program that addresses the potential loss of material due to corrosion in the inaccessible areas of their Mark I steel containment drywell shell for the period of extended operation.

The NRC staff issues LR-ISGs to facilitate timely implementation of the license renewal rule and to review activities associated with a license renewal application. The NRC staff will also incorporate the approved LR-ISG into the next revision of the license renewal guidance documents.

ADDRESSES: The NRC maintains an Agencywide Documents Access and Management System (ADAMS), which provides text and image files of NRC's public documents. These documents may be accessed through the NRC's Public Electronic Reading Room on the

Internet at <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by e-mail at pdr@nrc.gov.

FOR FURTHER INFORMATION CONTACT: Ms. Linh Tran, License Renewal Project Manager, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC, 20555-0001, telephone 301-415-4103 or by e-mail at Lint@nrc.gov.

SUPPLEMENTARY INFORMATION:

Attachment 1 to this Federal Register notice, entitled Staff Position and Rationale for the Final License Renewal Interim Staff Guidance—LR-ISG-2006-01: Plant-specific Aging Management Program for Inaccessible Areas of Boiling Water Reactor Mark I Steel Containment Drywell Shell contains the NRC staff's rationale for publishing the Final LR-ISG-2006-01. Attachment 2, entitled Final License Renewal Interim Staff Guidance—LR-ISG-2006-01: Plant-specific Aging Management Program for Inaccessible Areas of BWR Mark I Steel Containment Drywell Shell, contains the guidance for developing the plant-specific aging management program. The NRC staff approves this LR-ISG for NRC and industry use. The NRC staff will also incorporate the approved LR-ISG into the next revision of the license renewal guidance documents.

For the Nuclear Regulatory Commission.

Dated at Rockville, Maryland, this 16th day of November 2006.

Frank P. Gillespie,

Director, Division of License Renewal, Office of Nuclear Reactor Regulation.

Attachment 1—Staff Position and Rationale for the Final License Renewal Interim Staff Guidance—LR-ISG-2006-01: Plant-Specific Aging Management Program for Inaccessible Areas of BWR Mark I Steel Containment Drywell Shell

Staff Position

The NRC staff determined that a plant-specific aging management program (AMP) is needed to address the potential loss of material due to corrosion in the inaccessible areas of the Mark I steel containment drywell shell for the period of extended operation.

Rationale

The current license renewal guidance documents (LRGDs) do not provide sufficient guidance to address

² Copies of this Notice were sent this date by Internet electronic mail transmission to counsel for (1) USEC; and (2) the NRC Staff.

inaccessible areas of the Mark I steel containment drywell shell. Specifically, the inaccessible areas where the drywell shell is surrounded by a concrete structure with a narrow distance between the steel shell and the surrounding concrete inhibit visual inspection. Past operating experience in Mark I steel containments indicates that when water is discovered in the bottom outside areas of the drywell (for example in the sand-bed area), the most likely cause would be the water seeping through the space between the drywell shell and the shield concrete.

In addition, numerous requests for additional information (RAIs) were necessary on previous and current license renewal applications (LRAs) to obtain the information needed by the staff to perform its review. The purpose of this LR-ISG is to provide guidance on the information that should be provided in the LRA to reduce the number of RAIs issued to the applicants. Specifically, the staff has determined that a plant-specific aging management program (AMP) is needed to address the potential loss of material due to corrosion in the inaccessible areas of the Mark I steel containment drywell shell for the period of extended operation.

The drywell shell is a passive, long-lived structure subject to aging degradation. Pursuant to 10 CFR 54.21, the applicant must demonstrate that the effects of aging will be adequately managed so that the intended function will be consistent with the current licensing basis (CLB) for the period of extended operation.

Attachment 2—Final License Renewal Interim Staff Guidance—LR-ISG-2006-01: Plant-Specific Aging Management Program for Inaccessible Areas of Boiling Water Reactor Mark I Steel Containment Drywell Shell

Introduction

Line Item II.B1.1-2 of NUREG-1801, Volume 2, Revision 1, includes a provision for aging management of the Mark I steel containment drywell shells. However, the line item requires additional detail to address the inaccessible areas of the Mark I steel containment drywell shells. Specifically, the line item does not provide guidance when the distance between the steel drywell shell and the surrounding concrete structure is too small for the successful performance of visual examination.

All Mark I containment drywells are free-standing steel construction, except for Brunswick, Units 1 and 2. The Brunswick Mark I containment consists of a reinforced concrete drywell and a

reinforced concrete torus with a steel liner. A drywell shell is a free-standing steel structure with no concrete backing, whereas the steel liner of a drywell is a leak-tight membrane in direct contact with the concrete containment.

Historical Background

Information Notice (IN) 86-99, "Degradation of Steel Containments," dated December 8, 1986, described an event related to the degradation of the drywell shell at Oyster Creek Nuclear Generating Station. IN 86-99, Supplement 1, dated February 14, 1991, explained that the most likely cause of corrosion of the drywell shell in sand-pocket areas (near the bottom of the drywell) and in the spherical portion of the drywell at higher elevations, was the water in the gap between the drywell and the concrete shield. The source of water was noted as leakage through the seal between the drywell and the refueling cavity. The IN supplement noted that the stainless steel liners in the refueling cavity and equipment pool developed cracks along the perimeter of the liner plates where they were welded to embedded channels. The IN supplement also noted that ultrasonic testing (UT) discovered minor corrosion in the cylindrical portion of the drywell, and significant corrosion in the sand-bed region of the shell.

Discussion

Generic Letter (GL) 87-05, "Request for Additional Information-Assessment of Licensee Measures to Mitigate And/Or Identify Potential Degradation of Mark I Drywells," requested additional information regarding licensee actions to mitigate and/or identify potential degradation of boiling water reactor Mark I drywells. As a result, a number of licensees performed UT of their carbon steel drywell shells adjacent to the sand-bed region. In addition, many licensees established leakage monitoring programs for drain lines to identify leakage that may have resulted from refueling or spillage of water into the gap between the drywell and the surrounding concrete. UT performed as a result of GL 87-05 provided a set of data points to determine the drywell shell thickness that could be compared to the nominal fabrication thickness and the minimum thickness required to withstand the postulated loads. These UT measurements taken during the 1987-1988 time frame fall approximately near the mid-point of the current 40-year operating license period for most plants with Mark I steel containments.

The drywell shell is a passive, long-lived structure within the scope of

license renewal that is subject to aging degradation. Pursuant to 10 CFR 54.21, the applicant must demonstrate that the effects of aging will be adequately managed so that the intended function will be maintained consistent with the current licensing basis for the period of extended operation. On the basis of license renewal application reviews and industry operating experience, the NRC staff determined that a plant-specific aging management program (AMP) is needed to address the potential loss of material due to corrosion in the inaccessible areas of the Mark I steel containment drywell shell for the period of extended operation.

Recommended Action

In addressing Line Item II.B1.1-2 of NUREG-1801, Volume 2, Revision 1, applicants for license renewal for plants with a Mark I steel containment should perform an aging management review of the inaccessible areas of its containment drywell shell and provide a plant-specific aging management program that addresses the potential loss of material due to corrosion for the period of extended operation.

In conducting the aging management review and developing the plant-specific aging management program for the drywell shell, the applicant should consider the following recommended actions based upon plant design and operating experience:

(1) Develop a corrosion rate that can be reasonably inferred from past UT examinations or establish a corrosion rate using representative samples in similar operating conditions, materials, and environments. If degradation has occurred, provide a technical basis using the developed or established corrosion rate to demonstrate that the drywell shell will have sufficient wall thickness to perform its intended function through the period of extended operation.

(2) Demonstrate that UT measurements performed in response to GL 87-05 did not show degradation inconsistent with the developed or established corrosion rate.

(3) Where degradation has been identified in the accessible areas of the drywell, provide an evaluation that addresses the condition of the inaccessible areas for similar conditions, that is, the applicant should evaluate the acceptability of inaccessible areas when conditions exist in the adjacent accessible areas that could indicate the presence of or could result in degradation to such inaccessible areas.

(4) To assure that there are no circumstances that would result in degradation of the drywell, demonstrate

that moisture levels associated with accelerated corrosion rates do not exist in the exterior portion of the drywell shell, for example: (1) The sand pocket area drains and/or the refueling seal drains are monitored periodically; (2) the top of the sand pocket area is sealed to exclude water accumulation in the sand pocket area; and/or alarms are used to monitor regions for moisture/leakage.

(5) If moisture has been detected or suspected¹ in the inaccessible area on the exterior of the drywell shell or the source of moisture cannot be determined subsequent to root cause analyses:

(a) Include in the scope of license renewal any components that are identified as a source of moisture, if applicable, such as the refueling seal or cracks in the stainless steel liners of the refueling cavity pool walls, and perform an aging management review.

(b) Identify surface areas requiring examination by implementing augmented inspections for the period of extended operation in accordance with the American Society of Mechanical Engineers (ASME) Section XI IWE-1240 as identified in Table IWE-2500-1, Examination Category E-C.

(c) Use examination methods, that are in accordance with ASME Section XI IWE-2500, which specifies:

(i) surface areas accessible from both sides shall be visually examined using a VT-1 visual examination method,

(ii) surface areas accessible from one side only shall be examined for wall thinning using an ultrasonic thickness measurement method,

(iii) when ultrasonic thickness measurements are performed, one foot square grids shall be used, unless justified otherwise, and

(iv) ultrasonic measurements shall be used to determine the minimum wall thickness within each grid. The location of the minimum wall thickness shall be marked such that periodic reexamination of that location can be performed.

(d) Demonstrate through use of augmented inspections performed in accordance with ASME Section XI IWE that corrosion is not occurring, or that corrosion is progressing so slowly that the age-related degradation will not jeopardize the intended function of the drywell shell through the period of extended operation.

(6) If the intended function of the drywell shell cannot be demonstrated for the period of extended operation (i.e., wall thickness is less than the minimum required thickness), identify actions that will be taken as part of the aging management program to ensure that the integrity of the drywell shell will be maintained through the period of extended operation.

[FR Doc. E6-19838 Filed 11-22-06; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[EA-06-244]

In the Matter of Dairyland Power Cooperative and All Other Persons Who Seek or Obtain Access to Safeguards Information Described Herein; Order Imposing Fingerprinting and Criminal History Records Check Requirements for Access to Safeguards Information (Effective Immediately)

I

The Licensee, Dairyland Power Cooperative, holds a license issued in accordance with the Atomic Energy Act (AEA) of 1954, as amended, by the U.S. Nuclear Regulatory Commission (NRC or Commission), authorizing it to engage in an activity subject to regulation by the Commission. On August 8, 2005, the Energy Policy Act of 2005 (EPAct) was enacted. Section 652 of the EPAct amended Section 149 of the AEA to require fingerprinting and a Federal Bureau of Investigation (FBI) identification and criminal history records check of any person who is to be permitted to have access to Safeguards Information (SGI)¹. The NRC's implementation of this requirement cannot await the completion of the SGI rulemaking, which is underway, because the EPAct fingerprinting and criminal history records check requirements for access to SGI were immediately effective upon enactment of the EPAct. Although the EPAct permits the Commission by rule to except certain categories of individuals from the fingerprinting requirement, which the Commission has done (see 10 CFR 73.59, 71 FR 33989 (June 13, 2006)), it is unlikely that licensee employees or others are excepted from the fingerprinting requirement by the "fingerprinting relief" rule. Individuals relieved from

fingerprinting and criminal history records checks under the relief rule include Federal, State, and local officials and law enforcement personnel; Agreement State inspectors who conduct security inspections on behalf of the NRC; members of Congress and certain employees of members of Congress or Congressional Committees, and representatives of the International Atomic Energy Agency (IAEA) or certain foreign government organizations. In addition, individuals who have a favorably-decided U.S. Government criminal history records check within the last five (5) years, or individuals who have active federal security clearances (provided in either case that they make available the appropriate documentation), have satisfied the EPAct fingerprinting requirement and need not be fingerprinted again. Therefore, in accordance with Section 149 of the AEA, as amended by the EPAct, the Commission is imposing additional requirements for access to SGI, as set forth by this Order, so that affected licensees can obtain and grant access to SGI. This Order also imposes requirements for access to SGI by any person, from any person², whether or not a Licensee, Applicant, or Certificate Holder of the Commission or Agreement States.

II

The Commission has broad statutory authority to protect and prohibit the unauthorized disclosure of SGI. Section 147 of the AEA grants the Commission explicit authority to issue such Orders as necessary to prohibit the unauthorized disclosure of SGI. Furthermore, Section 652 of the EPAct amended Section 149 of the AEA to require fingerprinting and an FBI identification and criminal history records check of each individual who seeks access to SGI. In addition, no person may have access to SGI unless the person has an established need-to-know the information and satisfies the trustworthy and reliability requirements described in Attachment 2 to Order EA-06-243.

In order to provide assurance that the Licensee is implementing appropriate

¹ The term "suspected" refers to surface areas likely to experience accelerated degradation and aging as described in IWE-1241(a) of Section XI of the ASME Code. Specifically, typical locations are those areas exposed to standing water, repeated wetting and drying, persistent leakage, and those with geometries that permit water accumulation, condensation, and microbiological attack.

¹ Safeguards Information is a form of sensitive, unclassified, security-related information that the Commission has the authority to designate and protect under section 147 of the AEA.

² Person means (1) any individual, corporation, partnership, firm, association, trust, estate, public or private institution, group, government agency other than the Commission or the Department of Energy, except that the Department of Energy shall be considered a person with respect to those facilities of the Department of Energy specified in section 202 of the Energy Reorganization Act of 1974 (88 Stat. 1244), any State or any political subdivision of, or any political entity within a State, any foreign government or nation or any political subdivision of any such government or nation, or other entity; and (2) any legal successor, representative, agent, or agency of the foregoing.