

NUCLEAR REGULATORY COMMISSION**Request for a License to Import Radioactive Waste**

Pursuant to 10 CFR 110.70(C) "Public notice of receipt of an application," please take notice that the Nuclear Regulatory Commission has received the following request for an import license.

Copies of the request are available electronically through ADAMS and can be accessed through the Public Electronic Reading Room (PERR) link <http://www.nrc.gov/NRC/ADAMS/index.html> at the NRC Home page.

A request for a hearing or petition for leave to intervene may be filed within 30 days after publication of this notice in the **Federal Register**. Any request for hearing or petition for leave to intervene

shall be served by the requestor or petitioner upon the applicant, the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555; the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555; and the Executive Secretary, U.S. Department of State, Washington, DC 20520.

The information concerning this request follows.

NRC IMPORT LICENSE APPLICATION

Name of applicant, date of application, date received, application number, docket number	Description of material	End use	Country of origin
Eastern Technologies, Inc., Ashford, AL, February 3, 2006. February 3, 2006 IW016 11005602	Class A radioactive waste consisting of corrosion activation and mixed fission products (predominantly Co-60, Co-58 and Mn-54) as contaminants on used protective clothing and other items.	Laundering and decontamination of protective clothing and related products used at the Laguna Verde Nuclear Power Plant located in Mexico.	Mexico.

For the Nuclear Regulatory Commission.
Dated this 24th day of February 2006 at Rockville, Maryland.

Stephen Dembek,

Acting Director, Office of International Programs.

[FR Doc. 06-2094 Filed 3-3-06; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-354]

PSEG Nuclear LLC; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (NRC or the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-57 issued to PSEG Nuclear LLC (the licensee) for operation of the Hope Creek Generating Station located in Salem County, New Jersey.

The proposed amendment would relocate the primary containment penetration conductor overcurrent protective devices and the Class 1E isolation breaker overcurrent protective devices from the Technical Specifications to the Updated Final Safety Analysis Report.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended

(the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The proposed changes do not increase the probability of any previously evaluated accident. No safety function has been altered. The proposed changes relocate the Primary Containment Penetration Conductor Overcurrent Protective Devices Limiting Condition for Operation (LCO) and Class 1E Isolation Breaker Overcurrent Protective Devices LCO requirements from the TS [technical specifications] to the Hope Creek Generating Station Updated Final Safety Analysis Report (UFSAR). Relocation of the Primary Containment

Penetration Conductor Overcurrent Protective Devices LCO and Class 1E Isolation Breaker Overcurrent Protective Devices LCO requirements is consistent with the NRC Final Policy Statement on TS Improvements and 10 CFR 50.36. In part, the Final Policy Statement provides screening criteria to evaluate TS requirements for the purpose of relocation to other licensee-controlled documents. LCOs which do not meet any of the Final Policy Statement criteria and any 10 CFR 50.36(c)(2)(ii) criteria may be proposed for relocation. The Primary Containment Penetration Conductor Overcurrent Protective Devices LCO and Class 1E Isolation Breaker Overcurrent Protective Devices LCO requirements do not satisfy any of the Final Policy Statement screening criteria. The proposed changes do not affect any operational characteristic, function, or reliability of any structure, system, or component (SSC). Thus the consequences of accidents previously analyzed are unchanged between the existing TS requirements and the proposed changes.

Based upon the above, the proposed change will not involve a significant increase in the probability or consequences of an accident previously analyzed.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated in the UFSAR. No