

does not routinely edit comment submissions to remove such information before making the comment submissions available to the public or entering the comment into ADAMS.

II. Discussion

As directed in SRM-SECY-11-0089, “Options for Proceeding with Future Level 3 Probabilistic Risk Assessment (PRA) Activities,” the staff is conducting a full-scope multi-unit site Level 3 PRA (Level 3 PRA project) that addresses all internal and external hazards, all plant operating modes, and all reactor units, spent fuel pools (SFPs), and dry cask storage. The reference site for this study contains 2 four-loop Westinghouse pressurized water reactors with large dry containments. The objectives of the Level 3 PRA project are to (1) develop a Level 3 PRA, generally based on current state-of-practice methods, tools, and data, that (a) reflects technical advances since the last NRC-sponsored Level 3 PRAs (NUREG-1150), which were completed over 30 years ago, and (b) addresses

scope considerations that were not previously considered (e.g., low-power and shutdown risk, multi-unit risk, other radiological sources); (2) extract new insights to enhance regulatory decision making and to help focus limited NRC resources on issues most directly related to the agency’s mission to protect public health and safety; (3) enhance PRA staff capability and expertise and improve documentation practices to make PRA information more accessible, retrievable, and understandable; and (4) demonstrate technical feasibility and evaluate the realistic cost of developing new Level 3 PRAs.

The work performed under this project is being documented as a multi-volume report. The current Level 3 PRA project reports describe the analyses and results for the SFP combined Level 1 and Level 2 PRA for all hazards (Volume 6a) and the SFP Level 3 PRA for all hazards (Volume 6b). Consistent with previous SFP studies, seismic events were found to be the dominant contributors (particularly, the higher

seismic bins). However, effectively no populations were projected to receive the high doses needed to result in early health effects. The bulk of the radiological exposure (which is in the low to very low dose range) arises from reoccupation of land after decontamination and natural decay and weathering reduce exposures to levels below the habitability criteria assumed in the analysis. The study also found that the risk (which considers both frequency and consequences) of latent fatality for the SFPs is less than that for the reactors.

While these results can vary for other plants due to variations in their design and siting, the estimates derived for the reference plant, when adjusted for siting and design variations, would provide useful qualitative risk insights for other U.S. operating plants.

III. Availability of Documents

The documents identified in the following table are available to interested persons through ADAMS, as indicated.

Document description	ADAMS Accession No.
SRM-SECY-11-0089, “Options for Proceeding with Future Level 3 Probabilistic Risk Assessment (PRA) Activities,” dated September 21, 2011.	ML112640419
Level 3 PRA Project, Volume 6a: Spent Fuel Pool Level 1 and Level 2 PRA (draft for public comment), published June 2025	ML25134A014
Level 3 PRA Project, Volume 6b: Spent Fuel Pool Level 3 PRA (draft for public comment), published June 2025	ML25134A015

Dated: June 25, 2025.

For the Nuclear Regulatory Commission.

Michelle Gonzalez,

Acting Chief, Probability Risk Assessment Branch, Division of Risk Analysis, Office of Nuclear Regulatory Research.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-255-LA-4; ASLBP No. 25-988-01-LA-BD01]

HOLTEC PALISADES, LLC; ESTABLISHMENT OF ATOMIC SAFETY AND LICENSING BOARD

Pursuant to the Commission’s regulations, *see, e.g.*, 10 CFR 2.104, 2.105, 2.300, 2.309, 2.313, 2.318, 2.321, notice is hereby given that an Atomic Safety and Licensing Board (Board) is being established to preside over the following proceeding:

HOLTEC PALISADES, LLC (Palisades Nuclear Plant)

Holtec Palisades, LLC has requested an amendment to Renewed Facility

Operating License No. DPR-20 to allow the use of Framatome Alloy 690 sleeves to repair defective steam generator tubes, in support of the potential reauthorization of power operations at the Palisades Nuclear Plant, which is located in Covert Township, Michigan. In response to a notice filed in the **Federal Register** announcing the opportunity to request a hearing, *see* 90 FR 15,722 (Apr. 15, 2025), Beyond Nuclear, Don’t Waste Michigan, Michigan Safe Energy Future, Three Mile Island Alert, and Nuclear Energy Information Service filed a hearing request on June 16, 2025.

The Board is comprised of the following Administrative Judges:

Michael M. Gibson, Chair, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001

Dr. Gary S. Arnold, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001

Dr. Arielle J. Miller, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001

All correspondence, documents, and other materials shall be filed in accordance with the NRC E-Filing rule. *See* 10 CFR 2.302.

Dated: June 25, 2025.

Emily I. Krause,

Associate Chief Administrative Judge, Atomic Safety and Licensing Board Panel, Rockville, Maryland.

[FR Doc. 2025-11996 Filed 6-27-25; 8:45 am]

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POSTAL REGULATORY COMMISSION

[Docket Nos. MC2025-1539 and K2025-1533; MC2025-1540 and K2025-1534]

New Postal Products

AGENCY: Postal Regulatory Commission.
ACTION: Notice.

SUMMARY: The Commission is noticing a recent Postal Service filing for the Commission’s consideration concerning a negotiated service agreement. This notice informs the public of the filing, invites public comment, and takes other administrative steps.

DATES: *Comments are due:* July 2, 2025.