MATTERS TO BE CONSIDERED: 1. One (1) Creditor Claim Appeal. Closed pursuant to Exemptions (6) and (8).

FOR FURTHER INFORMATION CONTACT: Becky Baker, Secretary of the Board, Telephone: 703–518–6304.

Becky Baker,

Secretary of the Board

[FR Doc. 03–19305 Filed 7–24–03; 5:14 pm]

BILLING CODE 7535–01–M

NATIONAL TRANSPORTATION SAFETY BOARD

Sunshine Act Meeting

TIME AND DATE: 9:30 a.m., Tuesday, August 5, 2003.

PLACE: NTSB Conference Center, 429 L'Enfant Plaza, SW., Washington, D.C. 20594.

STATUS: The three items are Open to the Public.

MATTERS TO BE CONSIDERED:

7487A—Railroad Accident Report—Derailment of Amtrak Auto Train P052–18 on the CSXT Railroad near Crescent City, Florida, on April 18, 2002.

7575—Railroad Accident Report—Uncontrolled Movement, Collision and Passenger Fatality on the Angels Flight Railway in Los Angeles, California, on February 1, 2001.

7299A—Aviation Accident Report—Emery Worldwide Airlines, Inc., McDonnell Douglas DC–8–71F, N8079U, Rancho Cordova, California, on February 16, 2000.

News Media Contact: Telephone: (202) 314–6100.

Individuals requesting specific accommodations should contact Ms. Carolyn Dargan at (202) 314–6305 by Friday, August 1, 2003.

FOR FURTHER INFORMATION CONTACT: Vicky D'Onofrio, (202) 314–6410.

Dated: July 25, 2003.

Vicky D'Onofrio,

Federal Register Liaison Officer.

[FR Doc. 03–19324 Filed 7–25–03; 11:18 am]

BILLING CODE 7533–01–M

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–155 & 72–043]

Consumers Energy Co., Big Rock Point Nuclear Plant; Notice of Receipt, Availability for Comment, and Meeting to Discuss License Termination Plan

The Nuclear Regulatory Commission (NRC) is in receipt of, and is making available for public inspection and comment, the License Termination Plan (LTP) for the Big Rock Point Nuclear Facility (BRP) located in Charlevoix, Michigan.

Reactor operations at the BRP ended in August 29, 1997. The reactor was defueled and all fuel moved to an independent spent fuel storage installation (ISFSI) in March 2003. In accordance with NRC regulations in effect at that time, the licensee submitted a decommissioning plan for the BRP to the NRC in February 1995. When proposed amendments to the NRC's decommissioning regulations were published in the **Federal Register** on July 29, 1996 (61 FR 39278), the licensee requested that the review of the decommissioning plan be suspended. When the amended regulations became effective on August 28, 1996, the submitted decommissioning plan, as supplemented, became the BRP Post Shutdown Decommissioning Activities Report (PSDAR) pursuant to 10 CFR 50.82, as amended. A public meeting was held in Charlevoix, Michigan, on November 13, 1997, to provide information and gather pubic comment on the PSDAR. The facility is undergoing active decontamination and dismantlement.

In accordance with 10 CFR 50.82(a)(9), all power reactor licensees must submit an application for termination of their license. The application for termination of license must be accompanied by or preceded by an LTP submitted for NRC approval. If found acceptable by the NRC staff, the LTP is approved by license amendment, subject to such conditions and limitations as the NRC staff deems appropriate and necessary. The licensee submitted the proposed LTP for the BRP by application dated April 1, 2003. In accordance with 10 CFR 20.1405 and 10 CFR 50.82(a)(9)(iii), the NRC is providing notice to individuals in the vicinity of the site that the NRC is in receipt of the BRP LTP, and will accept comments from affected parties. In accordance with 10 CFR 50.82(a)(9)(iii), the NRC is also providing notice that the NRC staff will conduct a meeting to discuss the BRP LTP on Tuesday, August 5, 2003, at 7 p.m., at the Charlevoix Stroud Hall located at 12491 Waller Road, Charlevoix, Michigan 49720.

The BRP LTP and associated environmental report are available for public inspection at NRC's Public Document Room at NRC Headquarters, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852. These documents are available for public review through ADAMS, the NRC's electronic reading room, at: *http://www.nrc.gov/reading-rm/adams.html.*

Dated at Rockville, Maryland, this 21st day of July, 2003.

For the Nuclear Regulatory Commission.

Daniel M. Gillen,

Chief, Decommissioning Branch, Division of Waste Management, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 03–19215 Filed 7–28–03; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–327 and 50–328]

Tennessee Valley Authority; Sequoyah Nuclear Plant, Units 1 and 2, Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from title 10 of the Code of Federal Regulations (10 CFR) part 50, section 50.60 for Facility Operating License Nos. DPR–77 and DPR–79, issued to the Tennessee Valley Authority (TVA, the licensee), for operation of the Sequoyah Nuclear Plant (SQN), Units 1 and 2, located in Hamilton County, Tennessee. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would permit the use of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (B&PV) Code, Section XI Code Case N–640, ''Alternative Requirement Fracture Toughness for Development of P–T Limit Curves for ASME B&PV Code, Section XI, Code Case N–640,'' in lieu of 10 CFR 50, Appendix G, paragraph IV.A.2.b.

The regulation at 10 CFR part 50, section 50.60(a), requires, in part, that except where an exemption is granted by the Commission, all light-water nuclear power reactors must meet the fracture toughness requirements for the reactor coolant pressure boundary set forth in Appendix G to 10 CFR part 50. Appendix G of 10 CFR part 50 requires the establishment of pressure-temperature (P–T) limits for specific material fracture toughness requirements of the reactor coolant pressure boundary materials and mandates the use of the ASME B&PV Code, Section XI, Appendix G. The requirements in 10 CFR 50, Appendix

G, establish an adequate margin to brittle failure during normal operation, anticipated operational occurrences, and system hydrostatic tests.

ASME B&PV Code, Section XI, Code Case N–640 permits the use of an alternate reference fracture toughness curve for reactor pressure vessel materials for use in determining the P–T limits. ASME Code Case N–640 permits the use of alternate reference fracture toughness (*i.e.*, use of ''K_{IC} fracture toughness curve'' instead of ''K_{IA} fracture toughness curve,'' where K_{IC} and K_{IA} are ''Reference Stress Intensity Factors,'' as defined in ASME Code, Section XI, Appendices A and G, respectively) for reactor vessel materials in determining the P–T limits. Since the K_{IC} fracture toughness curve shown in ASME Code, Section XI, Appendix A, Figure A–2200–1, provides greater allowable fracture toughness than the corresponding K_{IA} fracture toughness curve of ASME Code, Section XI, Appendix G, Figure G–2210–1, using ASME Code Case N–640 to establish the P–T limits would be less conservative than the methodology currently endorsed by 10 CFR part 50, Appendix G. Therefore, an exemption to apply ASME Code Case N–640 is required.

The proposed action is in accordance with the licensee's application dated September 6, 2002, as supplemented by letter dated December 19, 2002 and June 24, 2003.

The Need for the Proposed Action

The proposed exemption is needed to allow the licensee to implement ASME Code Case N–640 in order to revise the method used to determine the P–T limits because continued use of the present method for determining P–T limits unnecessarily restricts the P–T operating window. The two primary benefits to the licensee from the use of Code Case N–640 are:

• Challenges to the operators would be reduced since the requirements for maintaining high-vessel temperature during pressure testing would be lessened.

• Enhanced personnel safety would result because of the lower temperatures which would exist during the conduct of inspections in primary containment.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that there are no significant environmental impacts associated with the use of the alternative analysis method to support the revision of the reactor coolant system P–T limits.

The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types or significant increase in the amounts of effluents that may be released offsite, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not have a potential to affect any historic sites. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Environmental Impacts of the Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (*i.e.*, the ''no-action'' alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any different resource than those previously considered in the Final Environmental Statement for SQN, dated February 13, 1974.

Agencies and Persons Consulted

On July 15, 2003, the staff consulted with the Tennessee State official, Ms. Elizabeth Flannagan, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of this environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated September 6, 2002, as supplemented by letter dated December 19, 2002. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, *http://www.nrc.gov/reading-rm/adams.html.* Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1–800–397–4209 or 301–415–4737, or by e-mail to *pdr@nrc.gov.*

Dated at Rockville, Maryland, this 23rd day of July 2003.

For The Nuclear Regulatory Commission.

Allen G. Howe,

Chief, Section 2, Project Directorate 2, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 03–19213 Filed 7–28–03; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–327 and 50–328]

Tennessee Valley Authority; Notice of Withdrawal of Application for Amendment to Facility Operating License

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Tennessee Valley Authority (the licensee) to withdraw its May 22, 2003, application for proposed amendments to Facility Operating License Nos. DPR–77 and DPR–79 for the Sequoyah Nuclear Plant, Units 1 and 2, in Hamilton County, Tennessee.

The proposed amendment would have revised the limiting condition for operation for Technical Specification (TS) Section 3.7.5, ''Ultimate Heat Sink.'' The licensee requested that the maximum emergency raw cooling water temperature requirement in TS 3.7.5.b be increased from 83 degrees Fahrenheit (°F) to 87 °F and that the minimum ultimate heat sink water elevation in TS 3.7.5.a be increased from 670 feet to 674 feet.

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on July 8, 2003 (68 FR 40719). However, by letter dated July 17, 2003, the licensee withdrew the proposed change.

For further details with respect to this action, see the application for amendment dated May 22, 2003, and the licensee's letter dated July 17, 2003,