Dated: August 15, 2000.

Matthew M. Crouch,

Advisory Committee Management Officer, National Aeronautics and Space Administration.

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NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice 00–094]

Centennial of Flight Commission

AGENCY: National Aeronautics and Space Administration.

ACTION: Notice of meeting.

SUMMARY: In accordance with the Federal Advisory Committee Act, Pub. L. 92–463, as amended, the National Aeronautics and Space Administration announces a meeting of the Centennial of Flight Commission.

DATES: Tuesday, September 12, 2000, 1 p.m. to 3:30 p.m.

ADDRESSES: Smithsonian National Air and Space Museum, 7th and Independence Avenue, SW, Director's Conference Room, 3rd Floor, Washington, DC 20560. Attendees must check in at the Information Desk to be cleared to the 3rd floor.

FOR FURTHER INFORMATION CONTACT: Ms. Beverly Farmarco, Code ZC, National Aeronautics and Space Administration, Washington, DC 20546, 202/358–1903.

SUPPLEMENTARY INFORMATION: The meeting will be open to the public up to the seating capacity of the room. The agenda for the meeting is as follows:

—Experimental Aircraft Association Plans for the Wright Centennial

—Discussion of Chair's Draft ''White Paper on a Strategy Posture for the US Centennial of Flight Commission''

—Discussion of FY 2000 Report to Congress

—Plans for Next Meeting

It is imperative that the meeting be held on this date to accommodate the scheduling priorities of the key participants. Visitors will be requested to sign a visitor's register.

Dated: August 15, 2000.

Matthew M. Crouch

Advisory Committee Management Officer, National Aeronautics and Space Administration.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50–412]

Pennsylvania Power Company, Ohio Edison Company, The Cleveland Electric Illuminating Company, The Toledo Edison Company, Firstenergy Nuclear Operating Company; Beaver Valley Power Station, Unit No. 2; Environmental Assessment and Finding of No Significant Impact

The Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from the requirements of title 10 of the Code of Federal Regulations (10 CFR) § 50.60(a), and 10 CFR part 50, Appendix G, for Facility Operating License No. NPF–73, issued to FirstEnergy Nuclear Operating Company (the licensee), for operation of the Beaver Valley Power Station, Unit No. 2 (BVPS–2), located in Beaver County, Pennsylvania.

Environmental Assessment

Identification of the Proposed Action

Appendix G to 10 CFR part 50, requires that pressure/temperature (P/T) limits be established for reactor pressure vessels during normal operating and hydrostatic or leak rate testing conditions. Specifically, this regulation states, ''The appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions.'' Additionally, it specifies that the requirements for these limits are contained in the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Appendix G.

To address provisions of an amendment to the Technical Specification P/T limits, the licensee requested in its submittal dated June 17, 1999, that the NRC staff exempt BVPS–2 from the requirements of 10 CFR part 50, § 50.60(a), and 10 CFR part 50, Appendix G, to allow application of ASME Code Case N–640 in establishing the reactor vessel pressure limits at low temperatures.

Code Case N–640 permits the use of an alternate reference fracture toughness (K_{IC} fracture toughness curve instead of the K_{Ia} fracture toughness curve) for reactor vessel materials in determining the P/T limits. Since the K_{IC} fracture toughness curve shown in ASME, Section XI, Appendix A, Figure A–2200–1 (the K_{IC} fracture toughness curve), provides greater allowable fracture toughness than the corresponding K_{Ia} fracture toughness curve of ASME, Section XI, Appendix G, Figure G–2210–1 (the K_{Ia} fracture toughness curve), using Code Case N–640 for establishing the P/T limits would be less conservative than the methodology currently endorsed by 10 CFR part 50, Appendix G. Therefore, an exemption is required in order to apply the Code Case. It should be noted that, although Code Case N–640 was incorporated into the ASME Code recently, an exemption is still required because the proposed P/T limits (excluding Code Case N–640) are based on the 1989 edition of the ASME Code.

The proposed action is in accordance with the licensee's application for exemption dated June 17, 1999.

The Need for the Proposed Action

ASME Code Case N–640 is needed to revise the method used to determine the reactor coolant system (RCS) P/T limits.

The purpose of 10 CFR part 50, § 50.60(a), and 10 CFR part 50, Appendix G, is to protect the integrity of the reactor coolant pressure boundary in nuclear power plants. This is accomplished through these regulations that, in part, specify fracture toughness requirements for ferritic materials of the reactor coolant pressure boundary. Pursuant to 10 CFR part 50, Appendix G, it is required that P/T limits for the RCS be at least as conservative as those obtained by applying the methodology of the ASME Code, Section XI, Appendix G.

Current overpressure protection system (OPPS) setpoints produce operational constraints by limiting the P/T range available to the operator to heat up or cool down the plant. The operating window through which the operator heats up and cools down the RCS becomes more restrictive with continued reactor vessel service. Reducing this operating window could potentially have an adverse safety impact by increasing the possibility of inadvertent OPPS actuation due to pressure surges associated with normal plant evolutions such as reactor coolant pump start and swapping operating charging pumps with the RCS in a water-solid condition. The impact on the P/T limits and OPPS setpoints has been evaluated for an increased service period to 15 effective full power years based on ASME Code, Section XI, Appendix G, requirements. The results indicate that OPPS would significantly restrict the ability to perform plant heatup and cooldown, create an unnecessary burden to plant operations, and challenge control of plant evolutions required with OPPS enabled. Continued operation of BVPS–2 with P/T curves developed to satisfy ASME Code, Section XI, Appendix G,