

on September 11 and 12, 2009. The notice provided an opportunity to submit comments on the NRC staff's proposed NSHC determination.

The supplemental letter dated September 8, 2009, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in *The Tribune*.

The Commission's related evaluation of the amendment, finding of exigent circumstances, consideration of public comments, state consultation, and final NSHC determination are contained in a safety evaluation dated September 17, 2009.

Attorney for licensee: Jennifer Post, Esq., Pacific Gas and Electric Company, P.O. Box 7442, San Francisco, California 94120.

NRC Branch Chief: Michael T. Markley.

Dated at Rockville, Maryland, this 25th day of September 2009.

For the Nuclear Regulatory Commission.

Joseph G. Giitter,

Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. E9-23780 Filed 10-5-09; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[NRC-2008-0361]

Notice of Availability for Comment of Draft Standard Review Plan for Renewal of Independent Spent Fuel Storage Installation Licenses and Dry Cask Storage System Certificates of Compliance

AGENCY: U.S. Nuclear Regulatory Commission.

ACTION: Notice of availability and opportunity to provide comments.

DATES: Comments must be provided by December 21, 2009.

FOR FURTHER INFORMATION CONTACT: Ata Istar, Structural Mechanics and Materials Branch, Division of Spent Fuel Storage and Transportation Division, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20005-0001. Telephone: (301) 492-3409; fax number: (301) 492-3342; e-mail: ata.istar@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Introduction

The Nuclear Regulatory Commission (NRC) has prepared a draft Standard Review Plan (SRP) NUREG-1927, entitled "Standard Review Plan for Renewal of Independent Spent Fuel Storage Installation Licenses and Dry Cask Storage System Certificate of Compliance." This draft SRP would provide guidance to the NRC staff when reviewing Safety Analyses Reports submitted by applicants for renewals of specific Independent Spent Fuel Storage Installation licenses or dry cask storage system certificates of compliance under 10 CFR part 72. This draft SRP is related to the proposed rule published in the **Federal Register** on September 15, 2009 (74 FR 47126). The NRC is soliciting public comments on this draft SRP, which will be considered before the NRC issues the final version.

II. Further Information

Documents related to this action are available electronically at the NRC's Electronic Reading Room at <http://www.nrc.gov/reading-rm/adams.html>. From this site, you can access the NRC's Agencywide Documents Access and Management System (ADAMS), which provides text and image files of NRC's public documents. The ADAMS accession numbers for the documents related to this notice are provided in the following table. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by e-mail to ata.istar@nrc.gov.

Interim staff guidance documents	ADAMS accession No.
Draft of SRP NUREG-1927.	ML092510340.

These documents may also be viewed electronically on the public computers located at the NRC's PDR, O-1 F21, One White Flint North, 11555 Rockville Pike, Rockville, MD 20852. The PDR reproduction contractor will copy documents for a fee. Comments and questions on this draft SRP should be directed to Ata Istar, Structural Mechanics and Materials Branch, Division of Spent Fuel Storage and Transportation, Office of Nuclear Materials Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20005-0001 by December 21, 2009. Comments received after this date will be considered if it is practical to do so, but assurance of consideration cannot be given to comments received after this date.

Comments can also be submitted by telephone, fax, or e-mail to the following: Telephone: (301) 492-3409; fax number: (301) 492-3342; e-mail: ata.istar@nrc.gov.

Dated at Rockville, Maryland, this 29th day of September 2009.

For the U.S. Nuclear Regulatory Commission.

Christopher M. Regan,

Chief, Structural Mechanics and Materials Branch, Division of Spent Fuel Storage and Transportation, Office of Nuclear Material Safety and Safeguards.

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-361 and 50-362; NRC-2009-0439]

Southern California Edison Company; San Onofre Nuclear Generating Station, Unit 2 and Unit 3; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of a temporary exemption from Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Section 50.46 and 10 CFR 50, Appendix K, for Facility Operating License Nos. NPF-10 and NPF-15, issued to Southern California Edison Company (SCE, the licensee), for operation of the San Onofre Nuclear Generating Station (SONGS), Unit 2 and Unit 3, respectively, located in San Diego County, California. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The requirements in 10 CFR 50.46 specifically, and 10 CFR 50, Appendix K implicitly, refer to the use of Zircaloy or ZIRLO cladding. Therefore, a temporary exemption is required to use fuel rods clad with an advanced zirconium-based alloy that is not either Zircaloy or ZIRLO. Unlike the current fuel assemblies, the lead fuel assemblies (LFAs) manufactured by AREVA NP will contain M5 alloy cladding material. The licensee has requested a temporary exemption to allow the use of M5 alloy cladding.

The temporary exemption would allow up to 16 LFAs manufactured by AREVA NP with M5 alloy cladding material to be inserted into the SONGS Unit 2 or Unit 3 reactor cores during the