

A.M.: Introductions and Updates—Office of Budget, Finance, and Award Management and Office of Information and Resource Management activities.

Presentation and Discussion—NSTC Subcommittee on Research Business Models; Information Technology Security.

P.M.: Presentation and Discussion—Emergency Preparedness: Meeting with NSF Deputy Director; Committee Discussion; Planning for next meeting; feedback; other business.

Dated: March 11, 2003.

Susanne Bolton,

Committee Management Officer.

[FR Doc. 03-6225 Filed 3-14-03; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Agency Information Collection Activities: Proposed Collection; Comment Request

AGENCY: Nuclear Regulatory Commission (NRC).

ACTION: Notice of pending NRC action to submit an information collection request to OMB and solicitation of public comment.

SUMMARY: The NRC is preparing a submittal to OMB for review of continued approval of information collections under the provisions of the Paperwork Reduction Act of 1995 (44 U.S.C. Chapter 35).

Information pertaining to the requirement to be submitted:

1. *The title of the information collection:* "Reports Concerning Possible Non-Routine Emergency Generic Problems".
2. *Current OMB approval number:* 3150-0012.
3. *How often the collection is required:* On occasion.
4. *Who is required or asked to report:* Nuclear power plant, non-power reactor, and materials applicants and licensees.
5. *The number of annual respondents:* 204 (104 reactor licensees; 100 material licensees).
6. *The number of hours needed annually to complete the requirement or request:* 53,680 (43,680 for reactor licensees and 10,000 for materials licensees).
7. *Abstract:* NRC is requesting approval authority to collect information concerning possible non-routine generic problems which would require prompt action from NRC to preclude potential threats to public health and safety.

Submit, by May 16, 2003, comments that address the following questions:

1. Is the proposed collection of information necessary for the NRC to properly perform its functions? Does the information have practical utility?

2. Is the burden estimate accurate?

3. Is there a way to enhance the quality, utility, and clarity of the information to be collected?

4. How can the burden of the information collection be minimized, including the use of automated collection techniques or other forms of information technology?

A copy of the draft supporting statement may be viewed free of charge at the NRC Public Document Room, One White Flint North, 11555 Rockville Pike, Room O-1 F23, Rockville, MD 20852. OMB clearance requests are available at the NRC worldwide web site: <http://www.nrc.gov/public-involve/doc-comment/omb/index.html>. The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions about the information collection requirements may be directed to the NRC Clearance Officer, Brenda Jo. Shelton, U.S. Nuclear Regulatory Commission, T-6 E6, Washington, DC 20555-0001, by telephone at 301-415-7233, or by Internet electronic mail at Infocollects@nrc.gov.

Dated at Rockville, Maryland, this 10th day of March 2003.

For the Nuclear Regulatory Commission.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 03-6285 Filed 3-14-03; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-327]

Tennessee Valley Authority; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (NRC or the Commission) is considering issuance of an amendment to Facility Operating License DPR-77 issued to the Sequoyah Nuclear Plant (SQN) for operation of Unit 1 located in Hamilton County, Tennessee.

The proposed amendment would revise the SQN, Unit 1, Updated Final Safety Analysis Report (UFSAR). The revision provides an alternative methodology using a Bar-Lock

Mechanical Splice in lieu of the Cadweld splice used in the original design and construction of the Unit 1 concrete shield building dome. This proposed Bar-Lock mechanical splice is described in Topical Report No. 24370-TR-C-001, "Alternate Rebar Splice—Bar-Lock Mechanical Splices," and is requested for implementation upon the restoration of the dome as part of the upcoming steam generator replacement project for SQN, Unit 1.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in title 10 of the Code of Federal Regulations (10 CFR), section 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

No. No changes in event classification, as discussed in UFSAR chapter 15, will occur due to use of the Bar-Lock couplers.

The restoration of the temporary concrete construction openings in the shield building will utilize Bar-Lock couplers to splice new rebar to the existing rebar. The shield building structure limits the release of radioactivity following an accident and protects the systems, structures, and components inside containment from external events. The accidents of interest are those that rely on the shield building to limit the release of radioactivity to the environment, and those that result from some external events. The design of the shield building is such that it is not postulated to fail and initiate an accident described in the UFSAR.

The Bar-Lock coupler qualification tests detailed in Topical Report 24370-TR-C-001 demonstrate that the Bar-Lock coupler meets the American Society of Mechanical Engineers (ASME) strength requirements and is, therefore, acceptable for use in nuclear safety-related applications. Based on these test results, it is concluded that use of the Bar-Lock couplers in restoring the temporary