

collaborative infrastructure to achieve systemic change. Collaborative infrastructure refers to the process by which partnering organizations come together to map out mutually reinforcing activities through: (1) Shared vision, (2) partnerships, (3) goals and metrics, (4) leadership and communication, and (5) expansion, sustainability and scale. Through these five design elements of collaborative infrastructure, the successful implementation of NSF INCLUDES will result in substantial advances toward a diverse, innovative, and well-prepared STEM workforce to support our Nation's economy and continued U.S. leadership in the global STEM enterprise. It is anticipated that NSF's investment will contribute to new and improved STEM career pathways, policies, opportunities to learn, and practices for equity and inclusion.

The initiative is supported by the NSF INCLUDES Coordination Hub (www.includesnetwork.org) that provides a framework for communication and networking, network assistance and reinforcement, and visibility and expansion for the NSF INCLUDES National Network as a whole. The Hub leads and supports the National Network, working to (1) facilitate the sharing of promising practices and data for broadening participation in STEM, (2) contribute to the knowledge base on broadening participation in STEM through research, and (3) establish a framework for communications and networking among partners, as well as across the National Network.

NSF is requesting OMB approval for the NSF INCLUDES Coordination Hub to collect information from members of the NSF INCLUDES National Network. The NSF INCLUDES Coordination Hub seeks to collect feedback data from Network members to help inform Hub activities, assess the development and health of the NSF INCLUDES National Network, and begin tracking progress against the Hub's theory of action for building a collaborative infrastructure at the Network level. The purpose of the collection is to allow Network members to provide feedback on Coordination Hub support to date and to identify support needs in the coming year and collect data that will inform the Hub's shared measures work and network support and expansion goals. This information will be used by the Hub to refine its activities in support of the Network and to share with Network members. The NSF INCLUDES National Network is composed of:

- NSF INCLUDES grantees
- Other NSF funded projects,

- Federal Coordination in STEM (FC-STEM) agencies,
- Scholars engaged in broadening participation research, and
- Organizations that support the development of talent from all sectors of society to build an inclusive STEM workforce.

Information collected will include name of the respondents, their affiliated organizations, email addresses, and home states. These personal identifiable information (PII) are collected primarily to categorize responses based on respondents' roles in the NSF INCLUDES National Network. These PII data will be accessed only by the Coordination Hub. Any public reporting of data will be in aggregate form, and any personal identifiers will be removed.

Use of the Information: The information collected is primarily for the use of the NSF INCLUDES Coordination Hub to understand the utility of the network in supporting their project success, and for informing design decisions the Coordination Hub will make regarding future programming and support provided to network members.

Estimate burden on the public: Estimated at 550 hours per year for the life of the Coordination Hub's cooperative agreement with NSF.

Respondents: Members of the NSF INCLUDES National Network. The NSF INCLUDES National Network is comprised of individuals who are interested in or working directly to broaden participation in STEM. Some of these individuals are NSF INCLUDES grantees; others who have received other NSF awards, or pursue broadening participation in STEM with support from other sources, including grants from federal, state, philanthropic, or business entities. Some are themselves representatives of these various types of funders or businesses, such as program officers at NSF, other Federal agencies, and private foundations.

Estimated number of respondents: 1,500.

Average Time per Reporting: 20 minutes.

Frequency: Once per year.

Comments: Comments are invited on: (a) Whether the proposed collection of information is necessary for the proper performance of the functions of the Agency, including whether the information shall have practical utility; (b) the accuracy of the Agency's estimate of the burden of the proposed collection of information; (c) ways to enhance the quality, utility, and clarity of the information on respondents, including through the use of automated

collection techniques or other forms of information technology; and (d) ways to minimize the burden of the collection of information on respondents, including through the use of automated collection techniques or other forms of information technology.

Please submit one copy of your comments by only one method. All submissions received must include the agency name and collection name identified above for this information collection. Commenters are strongly encouraged to transmit their comments electronically via email. Comments, including any personal information provided become a matter of public record. They will be summarized and/or included in the request for Office of Management and Budget approval of the information collection request.

Dated: October 6, 2020.

Suzanne H. Plimpton,

Reports Clearance Officer, National Science Foundation.

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BILLING CODE 7555-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-315 and 50-316; NRC-2020-0178]

Indiana Michigan Power Company; Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2

AGENCY: Nuclear Regulatory Commission.

ACTION: Exemption; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing an exemption in response to an April 7, 2020 request from Indiana Michigan Power Company (I&M, the licensee). The issuance of the exemption would permit I&M to align the regulatory requirements for reporting frequency with the current Final Safety Analysis Report update frequency for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2.

DATES: The exemption was issued on October 1, 2020.

ADDRESSES: Please refer to Docket ID NRC-2020-0178 when contacting the NRC about the availability of information regarding this document. You may obtain publicly available information related to this document using any of the following methods:

- *Federal Rulemaking website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2020-0178. Address questions about NRC docket IDs in *Regulations.gov* to Jennifer Borges;

telephone: 301-287-9127; email: Jennifer.Borges@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

• **NRC's Agencywide Documents Access and Management System (ADAMS):** You may obtain publicly available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document.

FOR FURTHER INFORMATION CONTACT: Scott P. Wall, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington DC 20555-0001; telephone: 301-415-2855; email: Scott.Wall@nrc.gov.

SUPPLEMENTARY INFORMATION: The text of the exemption is attached.

Dated: October 2, 2020.

For the Nuclear Regulatory Commission.

Scott P. Wall,

Senior Project Manager, Plant Licensing Branch III, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

Attachment—Exemption

NUCLEAR REGULATORY COMMISSION

Docket Nos. 50-315 and 50-316

Indiana Michigan Power Company

Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 Exemption

I. Background

The Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 (CNP), is a two-unit nuclear power plant located in Berrien County, Michigan. Indiana Michigan Power Company (I&M, the licensee) holds Renewed Facility Operating License Nos. DPR-58 and DPR-74 for CNP. These licenses are subject to the rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC, the Commission).

II. Request/Action

By letter dated April 7, 2020 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML20126G456), I&M requested an exemption from requirements of Title 10 of the *Code of Federal Regulations* (10 CFR) Section

54.37, "Additional records and recordkeeping requirements," and 10 CFR 50.54, "Conditions of licenses," specifically with respect to their references to 10 CFR 50.71, "Maintenance of records, making of reports," paragraph (e).

10 CFR 50.71(e)(4) states, in part, that "Subsequent revisions [to the final safety analysis report (FSAR)] must be filed annually or 6 months after each refueling outage provided the interval between successive updates does not exceed 24 months." The two CNP units share a common FSAR; therefore, this rule requires the licensee to update that same document within 6 months after a refueling outage for either unit. By letter dated March 3, 1998 (ADAMS Accession No. ML021090203), the NRC granted I&M an exemption from 10 CFR 50.71(e)(4) for CNP, which allowed the licensee to submit FSAR updates after each Unit No. 1 refueling outage, not to exceed 24 months between successive updates. This exemption was granted before renewed licenses were issued for CNP.

10 CFR 54.37(b) states, in part:

After the renewed license is issued, the FSAR update required by 10 CFR 50.71(e) must include any systems, structures, and components newly identified that would have been subject to an aging management review or evaluation of time-limited aging analyses in accordance with [10 CFR] 54.21.

10 CFR 50.54(a)(3) states, in part:

Changes to the quality assurance program description that do not reduce the commitments must be submitted to the NRC in accordance with the requirements of [10 CFR] 50.71(e).

The references in 10 CFR 54.37(b) and 10 CFR 50.54(a)(3) to 10 CFR 50.71(e) can be interpreted to include the reporting frequency prescribed in 10 CFR 50.71(e)(4). If interpreted in this way, 10 CFR 54.37(b) would require information related to newly identified systems, structures, and components at CNP that are subject to an aging management review or evaluation of time-limited aging analyses, and 10 CFR 50.54(a)(3) would require changes to the quality assurance program description that do not reduce the commitments, to be submitted to the NRC "annually or 6 months after each refueling outage provided the interval between successive updates does not exceed 24 months," despite the NRC's March 3, 1998 approval of an exemption from 10 CFR 50.71(e)(4) for CNP. The exemption that I&M now requests from 10 CFR 54.37(b) and 10 CFR 50.54(a)(3) would permit I&M to align the reporting frequency of these requirements with the CNP FSAR update frequency

permitted by the March 3, 1998 exemption.

III. Discussion

Pursuant to 10 CFR 50.12, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR part 50, which are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security and when any of the special circumstances listed in 10 CFR 50.12(a)(2) are present. These special circumstances include, among other things:

(a) Application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule and

(b) Compliance would result in undue hardship or other costs that are significantly in excess of those contemplated when the regulation was adopted, or that are significantly in excess of those incurred by others similarly situated.

10 CFR 54.15 states that exemptions from the requirements of 10 CFR part 54 may be granted by the Commission in accordance with 10 CFR 50.12.

A. Authorized by Law

The requested exemption from 10 CFR 54.37(b) and 10 CFR 50.54(a)(3) would permit I&M to align the reporting frequency of these requirements with the CNP FSAR update frequency permitted by the March 3, 1998 exemption. As stated above, 10 CFR 50.12 and 10 CFR 54.15 allow the NRC to grant exemptions from the requirements of 10 CFR parts 50 and 54 when the exemptions are authorized by law. The NRC staff has determined, as explained below, that granting the licensee's proposed exemption will not result in a violation of the Atomic Energy Act of 1954, as amended, or the Commission's regulations. Therefore, the exemption is authorized by law.

B. No Undue Risk to the Public Health and Safety

The underlying purpose of 10 CFR 50.71(e)(4) is to ensure that licensees periodically update their FSARs so that they accurately reflect the plant design and operation, which includes changes required pursuant to 10 CFR 54.37(b) and 10 CFR 50.54(a)(3). The NRC has determined by rule that a frequency not exceeding 24 months between successive updates is acceptable for maintaining FSAR content up-to-date. The requested exemption would provide an equivalent level of protection to the existing requirements

because it ensures that updates to the CNP FSAR are submitted with no greater than 24 months between successive updates. The requested exemption would also meet the intent of the rule with respect to regulatory burden reduction. Additionally, based on the nature of the requested exemption and the fact that updates will not exceed 24 months from the last submittal as described above, no new accident precursors would be created by the exemption; therefore, neither the probability nor the consequences of postulated accidents would be increased. In conclusion, the requested exemption will not present an undue risk to the public health and safety.

C. Consistent With the Common Defense and Security

The requested exemption from 10 CFR 54.37(b) and 10 CFR 50.54(a)(3) with respect to their references to 10 CFR 50.71(e) would allow I&M to continue to submit its periodic updates to the CNP FSAR within 6 months after each CNP, Unit No. 1 refueling outage, not to exceed 24 months from the last submittal. Neither these regulations nor the proposed exemption thereto has any relation to security issues. Therefore, the common defense and security is not impacted by the requested exemption.

D. Special Circumstances

Special circumstances, in accordance with 10 CFR 50.12(a)(2)(ii), are present whenever application of the regulation in the particular circumstances is not necessary to achieve the underlying purpose of the rule.

The underlying purpose of 10 CFR 54.37(b) and 10 CFR 50.54(a)(3) is to ensure that licensees periodically update their FSARs with changes required by these regulations so that the FSARs remain up-to-date and accurately reflect the plant design and operation. As previously described, the references in 10 CFR 54.37(b) and 10 CFR 50.54(a)(3) to 10 CFR 50.71(e) can be interpreted to include the reporting frequency prescribed in 10 CFR 50.71(e)(4). If interpreted in this way, strict compliance with the 10 CFR 54.37(b) and 10 CFR 50.54(a)(3) reporting requirements at CNP, where the FSAR is updated at a frequency permitted by the March 3, 1998 exemption, would create a disconnect between these report updates and the FSAR update. Specifically, since CNP is a dual-unit facility with a single shared FSAR and staggered refueling outages, application of the phrase "after each refueling outage" in 10 CFR 50.71(e)(4), as it relates to 10 CFR 54.37(b) and 10 CFR 50.54(a)(3), would result in more

frequent report updates than are necessary to achieve the underlying purpose of the rule. Therefore, special circumstances are present per 10 CFR 50.12(a)(2)(ii).

E. Environmental Considerations

With respect to its impact on the quality of the human environment, the NRC has determined that the issuance of the exemption discussed herein meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(25). Under 10 CFR 51.22(c)(25), the granting of an exemption from the requirements of any regulation of 10 CFR Chapter I (which includes 10 CFR 54.37 and 10 CFR 50.54) is an action that is a categorical exclusion, provided that:

- (i) There is no significant hazards consideration;
- (ii) There is no significant change in the types or significant increase in the amounts of any effluents that may be released offsite;
- (iii) There is no significant increase in individual or cumulative public or occupational radiation exposure;
- (iv) There is no significant construction impact;
- (v) There is no significant increase in the potential for or consequences from radiological accidents; and
- (vi) The requirements from which an exemption is sought involve:
 - (A) Recordkeeping requirements;
 - (B) Reporting requirements;
 - (C) Inspection or surveillance requirements;
 - (D) Equipment servicing or maintenance scheduling requirements;
 - (E) Education, training, experience, qualification, requalification or other employment suitability requirements;
 - (F) Safeguard plans, and materials control and accounting inventory scheduling requirements;
 - (G) Scheduling requirements;
 - (H) Surety, insurance or indemnity requirements; or
 - (I) Other requirements of an administrative, managerial, or organizational nature.

The NRC staff's determination that all of the criteria for this categorical exclusion are met is as follows:

I. 10 CFR 51.22(c)(25)(i): There is no significant hazards consideration.

Staff Analysis: The criteria for determining whether an action involves a significant hazards consideration are found in 10 CFR 50.92. The proposed action involves only a schedule change regarding the submission of an update to the application. Therefore, there are no significant hazards considerations because granting the exemption would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or

(2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or

(3) Involve a significant reduction in a margin of safety.

II. 10 CFR 51.22(c)(25)(ii): There is no significant change in the types or significant increase in the amounts of any effluents that may be released offsite.

Staff Analysis: The proposed action involves only a schedule change, which is administrative in nature, and does not involve any changes in the types or significant increase in the amounts of any effluents that may be released offsite.

III. 10 CFR 51.22(c)(25)(iii): There is no significant increase in individual or cumulative public or occupational radiation exposure.

Staff Analysis: Since the proposed action involves only a schedule change, which is administrative in nature, it does not contribute to any significant increase in individual or cumulative public or occupational radiation exposure.

IV. 10 CFR 51.22(c)(25)(iv): There is no significant construction impact.

Staff Analysis: Since the proposed action involves only a schedule change, which is administrative in nature, it does not involve any construction impact.

V. 10 CFR 51.22(c)(25)(v): There is no significant increase in the potential for or consequences from radiological accidents.

Staff Analysis: The proposed action involves only a schedule change, which is administrative in nature and does not impact the potential for or consequences from radiological accidents.

VI. 10 CFR 51.22(c)(25)(vi): The requirements from which the exemption is sought involve scheduling requirements and other requirements of an administrative, managerial, or organizational nature.

Staff Analysis: The proposed action involves scheduling requirements and other requirements of an administrative, managerial, or organizational nature because it is associated with the requirement in 10 CFR 50.71(e)(4), which stipulates that revisions to the FSAR must be filed annually or 6 months after each refueling outage provided the interval between successive updates does not exceed 24 months.

Based on the above, the NRC staff concludes that the proposed exemption meets the eligibility criteria for the categorical exclusion set forth in 10 CFR

51.22(c)(25). Therefore, in accordance with 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the NRC's issuance of this exemption.

IV. Conclusions

The NRC has determined that, pursuant to 10 CFR 50.12 and 10 CFR 54.15, the exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. Also, special circumstances are present. Therefore, the NRC hereby grants I&M an exemption from the requirements of 10 CFR 54.37(b) and 10 CFR 50.54(a)(3) with respect to their references to 10 CFR 50.71(e) to allow I&M to continue to submit its periodic updates to the CNP FSAR within 6 months after each CNP, Unit No. 1 refueling outage, not to exceed 24 months from the last submittal.

This exemption is effective upon issuance.

Dated: October 1, 2020.

For the Nuclear Regulatory Commission.

David J. Wrona,

Acting Deputy Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2020-22231 Filed 10-8-20; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-237 and 50-249; NRC-2020-0223]

Exelon Generation Company, LLC; Dresden Nuclear Power Station, Units 2 and 3

AGENCY: Nuclear Regulatory Commission.

ACTION: Exemption; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) has issued an exemption from regulatory requirements for Dresden Nuclear Power Station, Units 2 and 3, in response to a October 21, 2019, request from Exelon Generation Company, LLC in order to permit exclusion of main steam isolation valve (MSIV) leakage from the overall integrated leak rate Type A test measurement, and MSIV pathway leakage contributions from the combined leakage rate of all penetrations and valves subject to Type B and Type C tests.

DATES: The exemption was issued on October 5, 2020.

ADDRESSES: Please refer to Docket ID NRC-2020-0223 when contacting the NRC about the availability of information regarding this document. You may obtain publicly available information related to this document using any of the following methods:

- *Federal Rulemaking Website:* Go to <https://www.regulations.gov> and search for Docket ID NRC-2020-0223. Address questions about Docket IDs in *Regulations.gov* to Jennifer Borges; telephone: 301-287-9127; email: Jennifer.Borges@nrc.gov. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document.

FOR FURTHER INFORMATION CONTACT:

Russell S. Haskell, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone: 301-415-1129, email: Russell.Haskell@nrc.gov.

SUPPLEMENTARY INFORMATION: The text of the exemption is attached.

Dated: October 5, 2020.

For the Nuclear Regulatory Commission.

Russell S. Haskell,

Project Manager, Plant Licensing Branch III, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

Attachment—Exemption

NUCLEAR REGULATORY COMMISSION

Docket Nos. 50-237 and 50-249

Exelon Generation Company, LLC, Dresden Nuclear Power Station, Units 2 and 3

Exemption

I. Background

Exelon Generation Company, LLC (EGC, the licensee) is the holder of Facility Operating License Nos. DPR-19 and DPR-25, which authorize operation of the Dresden Nuclear Power Station, Units 2 and 3 (DNPS). The licenses provide, among other things, that the facilities are subject to the rules,

regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect. The facilities each consist of a boiling, light-water reactor located in Grundy County, Illinois.

II. Request/Action

In its letter dated October 21, 2019, as supplemented by letters dated May 6, 2020, and August 24, 2020 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML19294A304, ML20127H891, and ML20237F317, respectively), EGC requested a permanent exemption from the Part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR), Appendix J, Option B, Section III.A requirements in order to permit exclusion of main steam isolation valve (MSIV) leakage from the overall integrated leak rate Type A test measurement, and from Option B, Section III.B, requirements to permit exclusion of the MSIV pathway leakage contributions from the combined leakage rate of all penetrations and valves subject to Type B and Type C tests. EGC also requested a revision to Technical Specification (TS) 3.6.1.3, "Primary Containment Isolation Valves (PCIVs)," Surveillance Requirement (SR) 3.6.1.3.10, that would revise the single and combined MSIV leakage rate limits; an addition of a new TS 3.6.2.6, "Drywell Spray," to reflect the crediting of drywell spray for fission product removal; and a revision to TS 3.6.4.1, "Secondary Containment," SR 3.6.4.1.1, to address short-duration conditions during which the secondary containment pressure may not meet the SR pressure requirement at DNPS. The license amendment requests are addressed separately.

Under Part 50 of 10 CFR, paragraph 50.54(o), primary reactor containments for water-cooled power reactors are subject to the requirements of Appendix J to 10 CFR part 50. Appendix J specifies the leakage rate test requirements, schedules, and acceptance criteria for tests of the leak-tight integrity of the reactor containment and systems and components that penetrate the containment. Option B of 10 CFR 50, Appendix J, "Performance-Based Requirements," paragraph III.A, "Type A Test," requires, among other things, that the overall integrated leakage rate must not exceed the allowable leakage rate (L_a) with margin, as specified in the TSs. The overall integrated leakage rate is defined in 10 CFR part 50, Appendix J, as "the total leakage rate through all tested leakage paths, including containment welds, valves, fittings, and components that penetrate the containment system." This includes the