If attending this meeting, please enter through the One White Flint North building, 11555 Rockville Pike, Rockville, MD. After registering with security, please contact Mr. Theron Brown (Telephone 240–888–9835) to be escorted to the meeting room.

Dated: September 26, 2012.

Antonio Dias,

Technical Advisor, Advisory Committee on Reactor Safeguards.

[FR Doc. 2012–24383 Filed 10–2–12; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[NRC–2011–0296]

Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post-Accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants

AGENCY: Nuclear Regulatory Commission.

ACTION: Regulatory guide; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC or the Commission) is issuing a revision to Regulatory Guide (RG) 1.52, ''Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post-accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants.'' This guide applies to the design, inspection, and testing of air filtration and iodine adsorption units of engineered-safety-feature (ESF) atmosphere cleanup systems in light-water-cooled nuclear power plants.

ADDRESSES: Please refer to Docket ID NRC–2011–0296 when contacting the NRC about the availability of information regarding this document. You may access information related to this document, which the NRC possesses and are publicly available, using any of the following methods:

- *Federal Rulemaking Web site:* Go to *http://www.regulations.gov* and search for Docket ID NRC–2011–0296. Address questions about NRC dockets to Carol Gallagher; telephone: 301–492–3668; email: *Carol.Gallagher@nrc.gov*.
- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may access publicly available documents online in the NRC Library at *http://www.nrc.gov/reading-rm/adams.html.* To begin the search, select ''ADAMS Public Documents'' and then select ''*Begin Web-based ADAMS Search.''* For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to *pdr.resource@nrc.gov.* The ADAMS accession number for each document referenced in this notice (if that document is available in ADAMS) is provided the first time that a document is referenced. Revision 4 of Regulatory Guide 1.52 is available in ADAMS under Accession No. ML12159A013. The regulatory analysis may be found in ADAMS under Accession No. ML12159A538.
- *NRC's PDR:* You may examine and purchase copies of public documents at the NRC's PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

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FOR FURTHER INFORMATION CONTACT: Mekonnen Bayssie, Regulatory Guide Development Branch, Division of Engineering, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001; telephone: 301–251–7489; email: *Mekonnen.Bayssie@nrc.gov*.

SUPPLEMENTARY INFORMATION:

I. Introduction

The NRC is issuing a revision to an existing guide in the NRC's ''Regulatory Guide'' series. This series was developed to describe and make available to the public information such as methods that are acceptable to the NRC staff for implementing specific parts of the agency's regulations, techniques that the staff uses in evaluating specific problems or postulated accidents, and data that the staff needs in its review of applications for permits and licenses.

Revision 4 of RG 1.52 was issued with a temporary identification as Draft Regulatory Guide, DG–1274. This guide provides a method that the NRC considers acceptable to implement part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR), ''Domestic Licensing of Production and Utilization Facilities,'' Appendix A, ''General Design Criteria for Nuclear Power Plants,'' as it applies to the design, inspection, and testing of air filtration and iodine adsorption units of ESF atmosphere cleanup systems in light-water-cooled nuclear power plants. For the purposes of this guide, ESF atmosphere cleanup systems are those systems that are credited in the licensee's current design-basis accident (DBA) analysis, as described in the safety analysis report (SAR), or those systems that the licensee has described in the SAR as ESF atmosphere cleanup systems. This guide addresses ESF atmosphere cleanup systems, including the various components and ductwork, in the postulated DBA environment.

The NRC published the previous Revision 3 of this RG in June 2001. Since this publication, the American Society of Mechanical Engineers (ASME) Committee on Nuclear Air and Gas Treatment (CONAGT) has expanded the scope of equipment covered by ASME AG–1, ''Code on Nuclear Air and Gas Treatment.'' The NRC staff had previously endorsed earlier revisions of ASME–AG–1 in RG 1.52. The revision to ASME–AG–1 consolidated select requirements from ASME–N509, ''Nuclear Power Plant Air-Cleaning Units and Components,'' ASME–N510, ''Testing of Nuclear Air-Treatment Systems,'' and other documents previously endorsed by the NRC staff in RG 1.52. In addition, CONAGT has developed and published a new standard, ASME–N511–2007, ''Inservice Testing of Nuclear Air Treatment, Heating Ventilation and Air Conditioning Systems.'' This new standard provides comprehensive test and inspection requirements and is written to complement the expanded ASME–AG–1. Revision 4 of this RG is necessary to address these changes to the referenced industry standards.

II. Further Information

DG–1274 was published in the **Federal Register** on December 30, 2011 (76 FR 82323), for a 60-day public comment period. The public comment period closed on February 25, 2012. Public comments on DG–1274 and the staff responses to the public comments are available under ADAMS Accession No. ML12159A049.

III. Backfitting and Issue Finality

Issuance of this final regulatory guide does not constitute backfitting as defined in 10 CFR 50.109 (the Backfit Rule) and is not otherwise inconsistent with the issue finality provisions in 10 CFR Part 52. As discussed in the ''Implementation'' section of this regulatory guide, the NRC has no current intention to impose this regulatory guide on holders of current operating licenses or combined licenses.

This regulatory guide may be applied to applications for operating licenses and combined licenses docketed by the NRC as of the date of issuance of the final regulatory guide, as well as future applications for operating licenses and combined licenses submitted after the issuance of the regulatory guide. Such action does not constitute backfitting as

defined in 10 CRF 50.109(a)(1) and is not otherwise inconsistent with the applicable issue finality provision in 10 CFR Part 52, inasmuch as such applicants or potential applicants are not within the scope of entities protected by the Backfit Rule or the relevant issue finality provisions in Part 52.

Dated at Rockville, Maryland, this 26th day of Sept., 2012.

For the Nuclear Regulatory Commission.

Thomas H. Boyce,

Chief, Regulatory Guide Development Branch, Division of Engineering, Office of Nuclear Regulatory Research.

[FR Doc. 2012–24395 Filed 10–2–12; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[NRC–2012–0227]

Regulatory Guide 5.67, Material Control and Accounting for Uranium Enrichment Facilities Authorized To Produce Special Nuclear Material of Low Strategic Significance

AGENCY: Nuclear Regulatory Commission.

ACTION: Regulatory guide; withdrawal.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is withdrawing Regulatory Guide (RG) 5.67, ''Material Control and Accounting for Uranium Enrichment Facilities Authorized to Produce Special Nuclear Material of Low Strategic Significance.'' The guide is being withdrawn because it is no longer needed and more extensive guidance can be found in NUREG/CR–5734, ''Recommendations to the NRC on Acceptable Format and Content for the Fundamental Nuclear Material Control (FNMC) Plan Required for Low-Enriched Uranium Facilities'' which was issued in November 1991.

ADDRESSES: Please refer to Docket ID NRC–2012–0227 when contacting the NRC about the availability of information on this document. You may access information related to this document, which the NRC possesses and is publicly available, using the following methods:

- *Federal Rulemaking Web site:* Go to *http://www.regulations.gov* and search for Docket ID NRC–2012–0227. Address questions about NRC dockets to Carol Gallagher; telephone: 301–492–3668; email: *Carol.Gallagher@nrc.gov*.
- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may access publicly available documents online in the NRC Library at *http://www.nrc.gov/reading-rm/adams.html.* To begin the search, select ''*ADAMS Public Documents*'' and then select ''*Begin Web-based ADAMS Search.''* For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to *pdr.resource@nrc.gov.* The review of the withdrawal of RG 5.67 is available in ADAMS under Accession No. ML12110A280.
- *NRC's PDR:* You may examine and purchase copies of public documents at the NRC's PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

FOR FURTHER INFORMATION CONTACT: Glenn Tuttle, Office of Nuclear Material Safety and Safeguards, Division of Fuel Cycle Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, telephone: 301–492–3129; or by email at *Glenn.Tuttle@nrc.gov*.

SUPPLEMENTARY INFORMATION:

I. Introduction

The NRC is withdrawing RG 5.67 because its guidance is no longer needed. RG 5.67 was published in December 1993 to provide guidance to the Paducah and Portsmouth Gaseous Diffusion Plants (GDPs) to develop their material control and accounting (MC&A) programs under Title 10, *Code of Federal Regulations* (10 CFR) part 76. RG 5.67 was used in conjunction with the 10 CFR part 74 MC&A guidance in NUREG/CR–5734, ''Recommendations to the NRC on Acceptable Format and Content for the Fundamental Nuclear Material Control (FNMC) Plan Required for Low-Enriched Uranium Facilities'' to develop the FNMC plans for the two GDPs in the 1990s.

The NRC is withdrawing this regulatory guide because NUREG/CR–5734 is more comprehensive than RG 5.67 and is applicable to the Paducah GDP and other uranium enrichment facilities that have been licensed under 10 CFR part 70. Topics that are covered only briefly in RG 5.67 are covered in detail in NUREG/CR–5734. Therefore, the guidance in RG 5.67 is not needed.

II. Further Information

As discussed above, the guidance provided in RG 5.67 is no longer necessary and is addressed in more detail by NUREG/CR–5734. Regulatory guides may be withdrawn when their guidance no longer provides useful information.

Withdrawal of a regulatory guide means that the NRC staff no longer approves, as a generic matter, the guidance in the withdrawn regulatory guide. Therefore, a certificate holder or licensee who wishes to follow the guidance bears the responsibility of demonstrating, in the appropriate circumstance, that the guidance in the withdrawn regulatory guide is applicable to the certificate holder or licensee's specific situation. Current certificate holders or licensees who have included RG 5.67 in their licensing basis may continue to use it, and withdrawal of the guide does not affect their existing licensing documents or agreements. Changes to existing licenses (or 10 CFR part 76 certificates) must be accomplished in accordance with applicable NRC requirements.

Dated at Rockville, Maryland, this 24th day of September, 2012.

For the Nuclear Regulatory Commission.

Thomas H. Boyce,

Branch Chief, Regulatory Guide Development Branch, Division of Engineering, Office of Nuclear Regulatory Research.

[FR Doc. 2012–24282 Filed 10–2–12; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–029 and 72–31; NRC–2012–0229]

Yankee Atomic Electric Company; Yankee Rowe Independent Spent Fuel Storage Installation, Staff Evaluation; Exemption

1.0 Background

Yankee Atomic Electric Company (YAEC, the licensee) is the holder of Facility Operating License No. DPR–3 which authorizes possession of nuclear fuel under part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR). The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC or the Commission) now or hereafter in effect. Per 10 CFR part 72, Subpart K, a general license is issued for the storage of spent fuel in an Independent Spent Fuel Storage Installation (ISFSI) to persons authorized to possess or operate nuclear power reactors under 10 CFR part 50. Thus, YAEC also holds a 10 CFR part 72 general license for storage of spent fuel and greater than Class C waste at the Yankee Rowe ISFSI in Rowe, Massachusetts.

The Yankee Nuclear Power Station (YNPS) was a Pressurized Water Reactor in Rowe, Massachusetts, operated by the Yankee Atomic Electric Company (YAEC). By February 26, 1992, the reactor core was removed, and YNPS