

approved collection for which approval has expired.

Agency: Bureau of Labor Statistics.

Title: CPS Displaced Worker, Job Tenure, and Occupational Mobility Supplement.

OMB Number: 1220-0104.

Affected Public: Households.

Total Respondents: 58,000.

Frequency: Biennially.

Total Responses: 58,000.

Average Time Per Response: 8 minutes.

Estimated Total Burden Hours: 7,733 hours.

Total Burden Cost (capital/startup): \$0.

Total Burden Cost (operating/maintenance): \$0.

Comments submitted in response to this notice will be summarized and/or included in the request for Office of Management and Budget approval of the information collection request; they also will become a matter of public record.

Signed at Washington, DC, this 18th day of June, 2003.

Cathy Kazanowski,

*Chief, Division of Management Systems,
Bureau of Labor Statistics.*

[FR Doc. 03-16419 Filed 6-27-03; 8:45 am]

BILLING CODE 4510-24-P

NATIONAL SCIENCE FOUNDATION

Committee Management Renewals

The NSF management officials have responsibility for the advisory committee listed below have determined that renewing this committee for another two years is necessary and in the public interest in connection with the performance of duties imposed upon the Director, National Science Foundation (NSF), by 42 U.S.C. 1861 *et seq.* This determination follows consultation with the Committee Management Secretariat, General Services Administration.

Proposal Review Panel for Social and Economic Sciences, #10748.

Effective date for renewals is July 1, 2003. For more information, please contact Susanne Bolton, NSF, at (703) 292-7488.

Dated: June 25, 2003.

Susanne Bolton,

Committee Management Officer.

[FR Doc. 03-16479 Filed 6-27-03; 8:45 am]

BILLING CODE 7555-01-M

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-272]

PSEG Nuclear, LLC; Salem Nuclear Generating Station, Unit No. 1; Exemption

1.0 Background

PSEG Nuclear, LLC (PSEG or the licensee) is the holder of Facility Operating License No. DPR-70 which authorizes operation of the Salem Nuclear Generating Station (Salem), Unit No. 1. The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect.

The facility consists of a pressurized-water reactor located in Salem County in the State of New Jersey.

2.0 Request/Action

Title 10 of the *Code of Federal Regulations* (10 CFR), § 50.48 requires that all nuclear power plants licensed to operate prior to January 1, 1979, have a fire protection plan that satisfies 10 CFR Part 50, Appendix A, General Design Criterion (GDC) No. 3, "Fire Protection." Appendix R to 10 CFR Part 50, "Fire Protection Program for Nuclear Power Facilities Operating Prior to January 1, 1979," hereinafter referred to as "Appendix R," establishes fire protection requirements to satisfy GDC 3. Furthermore, 10 CFR 50.48(b) specifically requires that all nuclear power plants licensed to operate prior to January 1, 1979, implement fire protection features described in Appendix R, Section III.G.

Appendix R requires that, if a licensee cannot meet the separation criteria delineated in Section III.G.2, and if redundant trains of safe shutdown cables or equipment are in the same fire area, the licensee must implement the alternative shutdown requirements of Section III.G.3. Section III.G.3 requires that plants have a shutdown capability independent of the area where redundant trains are located, and further requires that the area with redundant trains have an automatic fire detection and fixed fire suppression system. In addition, a Federal court decision¹ has held that, if a licensee implements the requirements of Section III.G.3, the licensee must also comply with Section III.L of Appendix R. Section III.L requires that the licensee assume that offsite power is lost for a fire in a fire

area crediting Section III.G.3 alternative shutdown.

Salem, Unit No. 1, began power operations prior to January 1, 1979. Therefore, the technical requirements of Appendix R, including Sections III.G and III.L, are directly applicable to Salem, Unit No. 1.

By letter dated May, 1, 2002, as supplemented on August 15, 2002, PSEG requested an exemption from Appendix R, Section III.G.3, fixed suppression requirements for Fire Areas 1(2)-FA-AB-64B (Reactor Plant Auxiliary Building, 64' Elevation) and 1(2)-FA-AB-84C (11 and 21 Component Cooling Water (CCW) System pump and heat exchanger areas, 84' Elevation). In addition, the licensee requested an exemption from Appendix R, Section III.L.3, loss of offsite power requirements for Fire Areas 1(2)-FA-AB-64B and 1(2)-FA-AB-84B (Reactor Plant Auxiliary Equipment Area, 84' Elevation).

On July 20, 1989, the Commission granted an exemption from the technical requirements of Appendix R, Section III.G.2, to the extent that Fire Areas 1(2)-FA-AB-64B and 1(2)-FA-AB-84B are not protected by automatic fire suppression systems. However, because these areas are being reclassified as alternate shutdown (Section III.G.3) fire areas, this exemption supercedes the specific exemptions granted on July 20, 1989, for Fire Areas 1-FA-AB-64B and 1-FA-AB-84B.

3.0 Discussion

Pursuant to 10 CFR 50.12, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR part 50 when: (1) The exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security; and (2) when special circumstances are present. Special circumstances exist if the application of the technical requirements of 10 CFR part 50, is not necessary to achieve the underlying purpose of the regulation. The underlying purpose of Sections III.G.3 and III.L.3 of Appendix R to 10 CFR part 50 is to provide reasonable assurance that at least one means of achieving and maintaining safe shutdown conditions will remain available during, and after, any postulated fire.

Fire Area 1-FA-AB-64B

PSEG requested an exemption from the fixed suppression requirement of Appendix R, Section III.G.3, and the loss of offsite power requirement of

¹ *Connecticut Light and Power, et al., v. NRC*, 673 F.2d 525 (D.C. Cir. 1982).

Appendix R, Section III.L for Fire Area 1-FA-AB-64B. This area contains waste gas compressors, waste gas tanks, storage tank recirculation pumps, a laundry pump, a chemical drain tank pump, and holdup tanks and pumps.

Fire Area 1-FA-AB-64B is made up of multiple rooms that are separated by reinforced concrete barriers. Each room contains either a single component or groups of similar components. The area boundaries are also constructed of reinforced concrete. In-situ combustibles in the fire area consist of lubricating oil in pumps and motors, paper, electrical cabinet internals, and cable insulation. A fire detection system is installed throughout the area except in the Holdup Tank Rooms, No. 1 Pump Waste Monitor Hold-up Tank Pump Room, the Waste Evaporator Feed Pump Room, and in unused space.

Manual fire alarm stations are provided in the area, and, along with the fire detection system, annunciate in the Control Room. Manual fire suppression capability is provided in the form of portable fire extinguishers and manual hose stations. As previously stated, the Commission granted an exemption for the lack of a fixed suppression system on July 20, 1989.

Technical Evaluation for Fire Area 1-FA-AB-64B

Fire Area 1-FA-AB-64B credits alternative shutdown capability to ensure post-fire safe shutdown. Therefore, because the area does not have a fixed suppression system and offsite power is necessary to accomplish safe shutdown, the licensee does not meet the technical requirements of 10 CFR part 50, Appendix R, Sections III.G.3 and III.L.3.

In its letter dated May 1, 2002, PSEG stated that a fire in this area has the potential to result in a loss of the emergency diesel generators (EDGs). However, the fire area contains no cables or equipment that could cause the station to lose offsite power. Therefore, offsite power would not be adversely affected by a fire, and would be available to support safe shutdown activities. Alternative shutdown capability, independent of the fire area, is provided to ensure post-fire safe shutdown. Based on its review, the NRC staff agrees that the licensee would be able to accomplish required safe shutdown activities for a fire in this fire area using offsite power.

The fire area also has a low in-situ combustible loading with few potential ignition sources. Areas containing significant amounts of combustible materials also have a detection system to warn plant operators in the event of

a fire. If a fire were to ignite in this area, the low combustible loadings, the inherent protection offered by the concrete wall construction, and the limited propagation pathways, would restrict the size of the fire. Fire detectors in the area would alarm, and the site fire brigade would be able to extinguish the fire using manual equipment. The NRC staff agrees that an automatic fire suppression system is not necessary to control a fire with the configuration of this fire area. Control Room operators would be able to promptly detect a fire, and the station fire brigade would rapidly respond and extinguish a fire in this area.

Therefore, the NRC staff concludes that the lack of fire suppression specified in Section III.G.3, and the capability to accommodate a loss of offsite power required by Section III.L.3, in Fire Area 1-FA-AB-64B does not present an undue risk to the public health and safety, and is not necessary to achieve the underlying purpose of Sections III.G.3 and III.L.3 of Appendix R to 10 CFR Part 50.

Fire Area 1-FA-AB-84B

The licensee requested an exemption from the loss of offsite power requirement of Section III.L.3 of Appendix R. The fire area contains pumps, heat exchangers, tanks and control centers for the chemical and volume control, CCW, safety injection, containment spray, auxiliary feedwater, waste disposal, and spent pool cooling systems.

Fire Area 1-FA-AB-84B consists of multiple rooms separated by reinforced concrete barriers. Each room contains either a single component or groups of similar components. In-situ combustibles in the area consist of lubricating oil in pumps and motors, flammable liquids stored in cabinets, and cable insulation. The area contains few ignition sources and few paths for fire propagation.

As previously stated, the Commission granted an exemption for the partial fire detection system on July 20, 1989. Fire suppression is provided for the auxiliary feedwater pumps by automatically actuated pre-action sprinkler systems. Fire suppression is provided for the charging pump area by a wet pipe sprinkler system.

Technical Evaluation of Fire Area 1-FA-AB-84B

The licensee does not meet the technical requirements of 10 CFR Part 50, Appendix R, in that Fire Area 1-FA-AB-84B credits alternative shutdown capability, and offsite power is required to accomplish safe shutdown.

Although a fire in this area has the potential to result in a loss of power from the EDGs to the 4160 Volt vital buses, the fire area contains no cables or equipment that could cause the station to also lose offsite power. Consequently, offsite power would not be adversely affected by a fire, and would, thus, be available to support safe shutdown activities in lieu of the EDGs. Alternative shutdown capability, independent of the fire area, is provided through the Chemical and Volume Control System cross-tie from the opposite unit to ensure post-fire safe shutdown. Based on its review, the staff agrees that the licensee would be able to accomplish required safe shutdown activities for a fire in this fire area using offsite power.

Therefore, because the licensee would still be able to achieve safe shutdown, the NRC staff concludes that the lack of capability to accommodate a loss of offsite power required by Section III.L.3, in Fire Area 1-FA-AB-84B does not present an undue risk to the public health and safety, and is not necessary to achieve the underlying purpose of Section III.L.3 of Appendix R to 10 CFR Part 50.

Fire Area 1-FA-AB-84C

The licensee requested an exemption from the fixed suppression requirement of Section III.G.3 of Appendix R for Fire Area 1-FA-AB-84C. This area contains the CCW Pump and Heat Exchanger.

The fire area boundaries are constructed of reinforced concrete. The area has a low combustible material loading, consisting of lubricating oil for the CCW pump and motor and cable insulation. The area contains few fixed ignition sources.

An area-wide fire detection system is installed in the area. A manual fire alarm station is provided in the corridor outside the room. Both systems alarm in the Control Room. Manual fire suppression capability is provided by portable fire extinguishers and manual hose stations located in the corridor outside of the fire area. A fire in this area could affect several systems required for redundant safe shutdown. Therefore, alternative shutdown capability, independent of the fire area, is provided to ensure post-fire safe shutdown.

Technical Evaluation of Fire Area 1-FA-AB-84C

The licensee does not meet the technical requirements of 10 CFR Part 50, Appendix R, Section III.G.3, in that Fire Area 1(2)-FA-AB-84C credits alternative shutdown capability and

does not have a fixed fire suppression system.

The fire area boundaries consist of reinforced concrete. Alternative shutdown capability exists independent of the fire area to ensure that the plant can be safely shutdown for a fire in this area. The low combustible loadings, principally electrical cable insulation, would result in a fire of limited size with slow growth characteristics. In the event of a fire in the area, the fire detectors would alarm and the fire brigade would extinguish the fire using manual equipment. The NRC staff agrees that an automatic fire suppression system is not necessary to control a fire in this configuration in this fire area. Control Room operators could detect a fire, and the station fire brigade would rapidly respond and extinguish a fire in this area.

Therefore, based on its review, the NRC staff concludes that the lack of fixed fire suppression specified in Appendix R, Section III.G.3, in Fire Area 1-FA-AB-84C does not present an undue risk to the public health and safety, and is not necessary to achieve the underlying purpose of the rule.

NRC Staff's Conclusion

The staff examined the licensee's rationale to support the exemption requests and concludes that the fire protection measures implemented in Fire Areas 1-FA-AB-64B, 1-FA-AB-84B, and 1-FA-AB-84C provide reasonable assurance that at least one means of achieving and maintaining safe shutdown conditions will remain available during and after any postulated fire.

Therefore, the NRC staff concludes that, pursuant to 10 CFR 50.12(a)(2)(ii), there are special circumstances present, in that the application of Sections III.G.2. and III.L.3 of 10 CFR Part 50, Appendix R, is not necessary in order to achieve the underlying purpose of those regulatory provisions.

4.0 Conclusion

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a), the exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. Also, special circumstances are present. Therefore, the Commission hereby grants PSEG Nuclear, LLC an exemption from 10 CFR, Part 50, Appendix R, Section III.G.3, fixed suppression requirements for Fire Areas 1-FA-AB-64B and 1-FA-AB-84C; and 10 CFR, Part 50, Appendix R, Section III.L.3, loss of offsite power requirements for Fire

Areas 1-FA-AB-64B and 1-FA-AB-84B for Salem, Unit No. 1.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will not have a significant effect on the quality of the human environment (68 FR 22742).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 24 day of June 2003.

For the Nuclear Regulatory Commission.

John A. Zwolinski,

Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 03-16415 Filed 6-27-03; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 030-35596]

Notice of Finding of No Significant Impact and Availability of Environmental Assessment for License Amendment of Materials License No. 37-30605-01, [Protarga, Inc., Exton, PA]

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) is considering the issuance of a license amendment to Protarga, Inc., for Materials License No. 37-30605-01, to authorize release of its facility in Exton, Pennsylvania, for unrestricted use and has prepared an Environmental Assessment (EA) in support of this action in accordance with the requirements of 10 CFR part 51. Based on the EA, the NRC has concluded that a Finding of No Significant Impact (FONSI) is appropriate.

II. EA Summary

The purpose of the proposed action is to allow for the release of the licensee's Exton, Pennsylvania facility for unrestricted use. Protarga, Inc., has been authorized by NRC from December 20, 2000, to use radioactive materials for research and development purposes at the site. On April 3, 2003, Protarga, Inc., requested that NRC release the facility for unrestricted use. Protarga, Inc., has conducted surveys of the facility and determined that the facility meets the license termination criteria in subpart E of 10 CFR part 20.

III. Finding of No Significant Impact

The NRC staff has evaluated Protarga, Inc.'s request and the results of the surveys and has concluded that the completed action complies with the

criteria in Subpart E 10 CFR Part 20. The staff has prepared the EA (summarized above) in support of the proposed license amendment to terminate the license and release the facility for unrestricted use. On the basis of the EA, the NRC has concluded that the environmental impacts from the proposed action are expected to be insignificant and has determined not to prepare an environmental impact statement for the proposed action.

IV. Further Information

The EA and the documents related to this proposed action, including the application for the license amendment and supporting documentation, are available for inspection at NRC's Public Electronic Reading Room at <http://www.nrc.gov/reading-rm/adams.html> (ADAMS Accession Nos. ML031700667 and ML031080087. These documents are also available for inspection and copying for a fee at the Region I Office, 475 Allendale Road, King of Prussia, PA 19406. Any questions with respect to this action should be referred to Sattar Lodhi, Nuclear Materials Safety Branch 2, Division of Nuclear Materials Safety, Region I, 475 Allendale Road, King of Prussia, Pennsylvania, 19406, telephone (610) 337-5364, fax (610) 337-5269.

Dated at King of Prussia, Pennsylvania this 19th day of June 2003.

For the Nuclear Regulatory Commission.

John D. Kinneman,

Chief, Nuclear Materials Safety Branch 2, Division of Nuclear Materials Safety, Region I.

[FR Doc. 03-16414 Filed 6-27-03; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Governors' Designees Receiving Advance Notification of Transportation of Nuclear Waste

On January 6, 1982 (47 FR 596 and 47 FR 600), the Nuclear Regulatory Commission (NRC) published in the **Federal Register** final amendments to 10 CFR parts 71 and 73 (effective July 6, 1982), that require advance notification to Governors or their designees by NRC licensees prior to transportation of certain shipments of nuclear waste and spent fuel. The advance notification covered in part 73 is for spent nuclear reactor fuel shipments and the notification for part 71 is for large quantity shipments of radioactive waste (and of spent nuclear reactor fuel not covered under the final amendment to 10 CFR part 73).